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ABSTRACT

NAC is a computer code designed to predict the neutron-induced gamma ray radioactivity for a wide variety of composite materials. This code is a subset of the NAP code, and the code output has been altered to provide convenient analysis by experimenters. The NAC output includes the input data, a list of all reactions for each constituent element, and the end-of-irradiation disintegration rates for each reaction. The code also compiles a product isotope inventory containing the isotope name, the disintegration rate, the gamma-ray source strength, and the absorbed dose rate at 1 meter from an unshielded point source. The induced activity is calculated as a function of irradiation and decay times; the effect of cyclic irradiation can also be calculated.

NAC: NEUTRON ACTIVATION CODE

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SUMMARY

NAC is a computer program designed to predict the neutron-induced gamma ray radioactivity for a wide variety of composite materials. The unshielded induced radioactivity is calculated as a function of neutron exposure and decay times. The effects of cyclic exposure to a neutron flux and target atom burn-up can also be calculated.

NAC, a subset of NAP, provides fewer computational options than NAP and the output has been altered to provide for more convenient analysis by experimenters. The NAC output includes the input data, a list of all reactions for each constituent element, and the end of irradiation disintegration rates for each reaction. The code also compiles a product isotope inventory containing the isotope name, the disintegration rate, the gamma ray source strength, and the absorbed dose rate at one meter from an unshielded point source. A breakdown of the above data as a function of gamma energy for each product isotope is available as an option.

INTRODUCTION

NAC is a computer code written to provide a means of rapid analysis of the neutron-induced gamma ray radioactivity for a wide variety of composite materials. The code is a simplified version of the NAP program (see ref. 1). NAP was written to calculate the induced activity and unshielded gamma dose as a function of time, space, and gamma energy. NAP was designed to use the neutron spectrum output from a transport code and to provide input data for a gamma shielding code. NAP is highly versatile and well suited to this type of activation calculation. However, the detailed calculations available with NAP are not warranted in experimental situations in which knowledge of the neutron spectrum is limited and the activation information required is minimal. Such a situation would occur

when the knowledge of the post-irradiation activation hazards of an experimental capsule, to be irradiated in a test reactor, is desired.

NAC was written to provide this type of activation calculation, and the code output has been designed to provide for convenient analysis by experimenters. The output consists of two sections. The first contains the input data (material composition, neutron fluxes, irradiation time, a list of the reactions considered for each element, the end of irradiation disintegration rates (dis/sec) for each reaction, and the fraction of the activity produced by each neutron energy group. The second section is a product isotope inventory containing the product name, disintegration rate (mCi), gamma source strength (MeV/sec), and the absorbed dose rate (m rads (C)/hr) at one meter from an unshielded point source. A breakdown of the above data as a function of gamma energy for each product isotope is available as an option. This output section also includes the decay time considered and totals of the inventory data. Simple scanning of the output will pin-point the product (s) which presents the greatest hazard and the reaction (s) which produces this product (see Appendix C).

The induced activity is calculated as a function of the duration of neutron exposure and the decay times. The effects of cyclic neutron exposure and of target atom burn-up can also be evaluated. The activity is calculated per-unit volume, per-unit mass, or for the total mass of the composite material, depending on the input specification.

ACTIVATION EQUATIONS

The build-up and decay of neutron-induced radioactivity are calculated from the equations given below which have been derived from the basic activation equations (see ref. 2). In each of the equations listed below, the term $\sigma\phi$ represents a summation over the entire neutron energy range and is independent of time.

For an irradiation time t_1 , with zero decay time, where target atom burn-up is ignored, the induced activities are:

for the parent

$$S_1(t_1) = N\sigma\varphi\left(1 - e^{-\lambda_1 t_1}\right)$$

for the daughter

$$S_2(t_1) = \frac{N\sigma\varphi}{(\lambda_2 - \lambda_1)} \left[\lambda_2 \left(1 - e^{-\lambda_1 t_1}\right) - \lambda_1 \left(1 - e^{-\lambda_2 t_1}\right) \right] \quad (1)$$

for the granddaughter

$$S_3(t_1) = \frac{N\sigma\varphi}{(\lambda_2 - \lambda_1)} \left[(\lambda_2 - \lambda_1) \left(1 - e^{-\lambda_3 t_1}\right) - \frac{\lambda_2 \lambda_3}{(\lambda_3 - \lambda_1)} \left(e^{-\lambda_1 t_1} - e^{-\lambda_3 t_1}\right) \right. \\ \left. + \frac{\lambda_1 \lambda_3}{(\lambda_3 - \lambda_2)} \left(e^{-\lambda_2 t_1} - e^{-\lambda_3 t_1}\right) \right]$$

where S_1 , S_2 , S_3 are the disintegration rates (dis/sec), N is the initial number of target atoms (number), σ is the activation cross section ($\text{cm}^2/\text{neutron}$), φ is the neutron flux (neutrons/ $\text{cm}^2\text{-sec}$), $\lambda_1, \lambda_2, \lambda_3$ are the decay constants for the parent, daughter, and granddaughter, respectively (sec^{-1}).

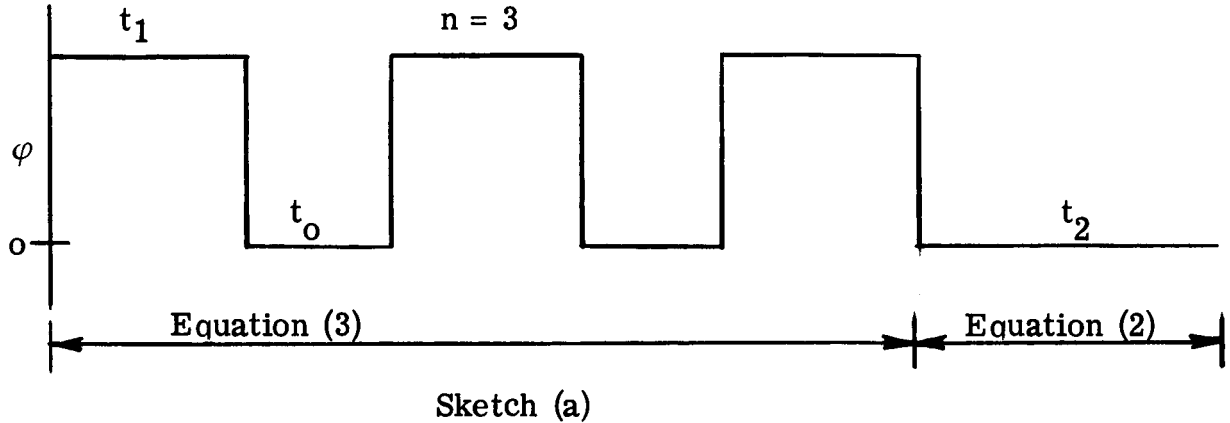
For a neutron irradiation time t_1 followed by a decay time t_2 , the induced activities are

$$\left. \begin{aligned} S_1(t_1, t_2) &= S_1(t_1)e^{-\lambda_1 t_2} \\ S_2(t_1, t_2) &= S_2(t_1)e^{-\lambda_2 t_2} + \frac{\lambda_2 S_1(t_1)}{(\lambda_2 - \lambda_1)} \left(e^{-\lambda_1 t_2} - e^{-\lambda_2 t_2} \right) \\ S_3(t_1, t_2) &= S_3(t_1)e^{-\lambda_3 t_2} + \frac{\lambda_3 S_2(t_1)}{\lambda_3 - \lambda_2} \left(e^{-\lambda_2 t_2} - e^{-\lambda_3 t_2} \right) \\ &\quad + \frac{\lambda_2 \lambda_3 S_1(t_1)}{(\lambda_2 - \lambda_1)} \left(\frac{e^{-\lambda_1 t_2} - e^{-\lambda_3 t_2}}{\lambda_3 - \lambda_1} - \frac{e^{-\lambda_2 t_2} - e^{-\lambda_3 t_2}}{\lambda_3 - \lambda_2} \right) \end{aligned} \right\} (2)$$

For a cyclic neutron exposure, with zero decay time following the end of the last exposure, the induced activities, adjusted for target atom burnup, are:

$$\left. \begin{aligned} S_1 &= \frac{N\sigma\phi\lambda_1}{\lambda_1 - \sigma\phi} \left(e^{-\sigma\phi t_1} - e^{-\lambda_1 t_1} \right) \left[\frac{e^{-n\sigma\phi t_1} - e^{-n\lambda_1(t_1 + t_0)}}{e^{-\sigma\phi t_1} - e^{-\lambda_1(t_1 + t_0)}} \right] \\ S_2 &= \frac{N\sigma\phi\lambda_1\lambda_2}{\lambda_1 - \sigma\phi} \left(\frac{e^{-\sigma\phi t_1} - e^{-\lambda_2 t_1}}{\lambda_2 - \lambda_1} \right) \left[\frac{e^{-n\sigma\phi t_1} - e^{-n\lambda_1(t_1 + t_0)}}{e^{-\sigma\phi t_1} - e^{-\lambda_1(t_1 + t_0)}} \right] \\ S_3 &= \frac{N\sigma\phi\lambda_1\lambda_2\lambda_3}{\lambda_1 - \sigma\phi} \left[\left(\frac{1}{\lambda_2 - \sigma\phi} \right) \left(\frac{e^{-\sigma\phi t_1} - e^{-\lambda_3 t_1}}{\lambda_3 - \sigma\phi} - \frac{e^{-\lambda_2 t_1} - e^{-\lambda_3 t_1}}{\lambda_3 - \lambda_2} \right) - \left(\frac{1}{\lambda_2 - \lambda_1} \right) \right. \\ &\quad \left. \times \left(\frac{e^{-\lambda_1 t_1} - e^{-\lambda_3 t_1}}{\lambda_3 - \lambda_1} - \frac{e^{-\lambda_2 t_1} - e^{-\lambda_3 t_1}}{\lambda_3 - \lambda_2} \right) \right] \left[\frac{e^{-n\sigma\phi t_1} - e^{-n\lambda_1(t_1 + t_0)}}{e^{-\sigma\phi t_1} - e^{-\lambda_1(t_1 + t_0)}} \right] \end{aligned} \right\} (3)$$

where n is the number of cycles, t_1 is the irradiation time per cycle, and t_0 is the non-irradiation time per cycle. Note that equations (3) calculate the end-of-exposure activity for n cycles (n irradiation and $(n-1)$ non-irradiation periods) with burn-up. The results of these equations are then used in equations (2) to calculate the activity after decay periods following cyclic exposure. See sketch (a).



The above equations yield the disintegration rates in disintegrations per second which are then converted to the desired output quantities. The gamma ray source strength (MeV/sec) is obtained by multiplying the disintegration rate by the appropriate gamma ray energies and the fraction of gamma rays emitted at each energy per disintegration. The absorbed dose rate \dot{D} in m rads (C)/hr at one meter from an unshielded point source is calculated from

$$\dot{D} = \frac{K}{4\pi R^2} S(\text{MeV/sec}) = 1.285 \times 10^{-8} S \quad (4)$$

where K is an energy flux to dose conversion factor, R is the distance from the source (1 m), and S is the source strength. K consists of a unit conversion factor and the mass energy absorption coefficient for carbon, which was arbitrarily selected as the 1 MeV value ($0.0280 \text{ cm}^2/\text{g}$). See reference 2. Appendix A is a complete FORTRAN listing of the NAC code.

PROGRAM DESCRIPTION

The NAC data library (see appendix B) contains the activation constants for 71 naturally occurring elements with a total of 251 reactions producing 226 radioactive isotopes. For each material to be analyzed, up to 20 different elements may be specified. Provision has been made for the analysis of as many different materials, per computer run, as desired. For each analysis, up to 20 decay time intervals may be specified. The NAC data library contains the data listed below for 71 naturally occurring elements.

- (1) target element names
- (2) isotopic reaction
- (3) activation cross sections (cm^2/g)
- (4) decay constants (min^{-1})
- (5) atomic densities (atoms/g)
- (6) product isotope name
- (7) decay gamma energies (MeV)
- (8) fraction of gammas at a given energy

The elements required for a material analysis are identified, during input, by their atomic number (Z) and are listed in the data library in order of increasing Z. If an element not present in the data library is specified during input, the code will write a message identifying the element in question and then will eliminate it from the calculation. Data input for each material must also include the material density or mass, the weight fractions of the constituent elements, the irradiation and decay time, and the neutron flux in the four energy groups listed below:

- | | |
|---------|--|
| Group 1 | $0.82 \text{ MeV} < E$ |
| Group 2 | $5.5 \text{ KeV} < E < 0.82 \text{ MeV}$ |
| Group 3 | $1.1 \text{ eV} < E < 5.5 \text{ KeV}$ |
| Group 4 | $E < 1.1 \text{ eV}$ |

The cross sections, taken from references 1 and 3, have been averaged as follows: for $0 < E \leq 0.2 \text{ eV}$, a Maxwellian distribution was used; for $0.2 \text{ eV} < E \leq 0.82 \text{ MeV}$, a $1/E$ distribution was used; and for $E > 0.82 \text{ MeV}$, the U^{235} fission spectrum was used.

PROGRAM OUTPUT

The program output contains the following data in the order listed.

Appendix C contains the output for 3 sample problems.

- item 1: information provided by the user to identify each material;
- item 2: the neutron flux in order of decreasing energy and the irradiation time, if the cycling option is used this is the irradiation time per cycle
- item 3: messages, if any, for elements requested which are not present in the library.
- item 4: element name and weight fraction
- item 5: list of reactions for the element; each followed by the disintegration rates (dis/sec) for the parent, daughter, and grand-daughter at zero decay time; and the fraction of induced activity due to each neutron group in order of decreasing energy.

Items 4 and 5 are repeated for all elements in a given material. The above information appears only once for each material. The following information is repeated for each decay time.

- item 6: the time after irradiation in minutes and the units in which the output is calculated: per-gram, per-cubic centimeter, or for the total mass.
- item 7: the product isotope name and total disintegration rate, source strength, and absorbed dose rate.

The following three lines appear if the breakdown of the activity as a function of gamma energy is desired.

- item 8: the gamma ray energies
- item 9: the absorbed dose rate at each gamma energy
- item 10: the source strength at each gamma energy.

The last item (11) appears regardless of the options used:

- 11: the total material disintegration rate, source strength, and absorbed dose rate.

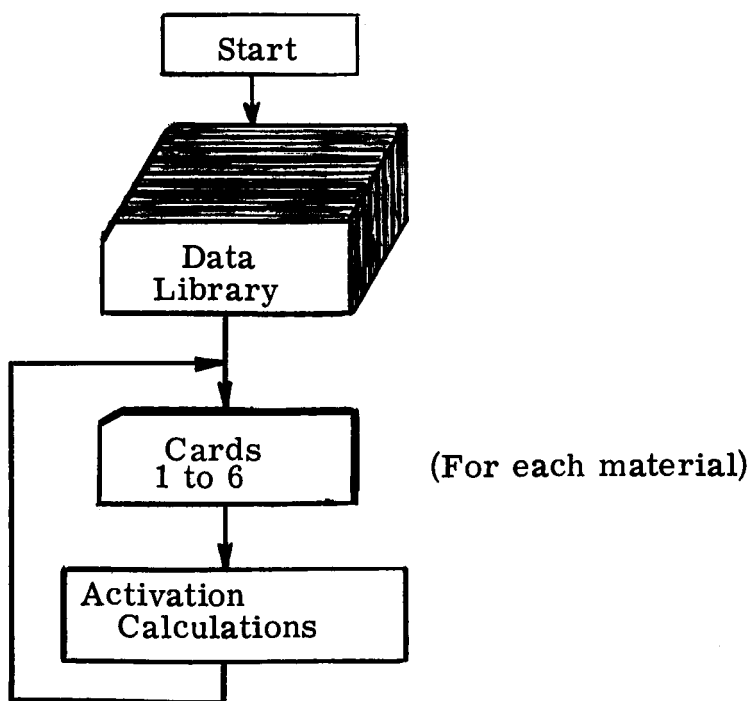
Items 6 through 11 are repeated for each time after irradiation. Those product activities which are zero (less than 10^{-38}) are not printed out.

PROGRAM LIMITATIONS

Each composite material may consist of up to 20 different elements and up to 20 different decay times may be included. The number of different materials that can be analyzed during a single computer "run" is limited only by the machine time available. All reactions for each element for which cross sections were available are included in the NAC library. These include the (n, γ) , (n, p) , (n, α) , and $(n, 2n)$ reactions; the formation of excited states and isomeric states are also included. The activity for each reaction is traced through the first three generations or until a stable product is formed. However, there are some exclusions: reactions which have products having half-lives less than about one minute and products which are not gamma emitters. The dose rate contributions of alpha and beta particles are not included.

INPUT PREPARATION

Each "data link" consists of one material to be analyzed. The library data deck must precede the first data link (see sketch (b)). The number of data links that can be processed per computer run is not limited; but all the cards listed below must be present for each link.



Sketch (b)
Input Flow

Card no.	Columns	Data
1	FORMAT (12A6) 1-72	any information desired by user to identify the material
2	FORMAT (22I3) 3 5, 6 7-66	= 1, if the gamma energy breakdown is not desired = 2, if the gamma energy breakdown is desired the number of elements desired; the Z value for each element desired;
3	FORMAT (8F10. 4) 1-10 11-80	= 1.0, for output per gram = density (g/cm^3) for output per cm^3 = - mass, for output per total mass the weight fraction of each element in the same order as the Z values on card 2
4	FORMAT (4E10, 4) 1-40	neutron flux in order of decreasing in energy;
5	FORMAT (I3, 2E10. 4, I3) 1-3 4-13 14-23 24-26	number of reactor cycles; columns are left blank if cycling is not desired irradiation time in minutes (if cycling is desired, the irradiation time per cycle) non-irradiation time per cycle in minutes; columns are left blank if cycling is not desired the number of decay times desired
6	FORMAT (8E10. 4) 1-80	the decay times in minutes from the end of irradiation (from the end of the last exposure for cycling)

Note that card five must be used with or without cycling. Normal (non-cyclic) calculations (equations (1)) do not include target atom burn-up. Calculation of target burn-up without cycling may be obtained by setting the number of cycles to 1 and the non-irradiation time per cycle to zero on card 5.

APPENDIX A A COMPLETE FORTRAN LISTING OF THE NAC CODE

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      INTEGER OPT,Z(21),PA(251),PB(251),PC(251)
      REAL LA(251),LB(251),LC(251)
      DIMENSION ELEM(2,71),REAC(5,251),SIGMA(4,251),ISO(2,226),NE(226),
      AEGAM(7,226),FRAC(7,226),IE(71),IDEL(71),AD(71),NAME(12),WTF(20),
      BPHI(4),T(20),FN(4),PROD(50,20),IP(50),SF(4),SFX(4),S(7),R(7),DF(4)
      C,DFN(4)
C
C      READ IN DATA LIBRARY
C
      READ (5,100)((ELEM(I,K),I=1,2),K=1,71)
100  FORMAT (12A6)
      READ (5,101) (IE(J),IDEL(J),J=1,71)
101  FORMAT (20I4)
      READ (5,102) (AD(J),J=1,71)
102  FORMAT (8E10.4)
      READ (5,103) ((REAC(I,J),I=1,5),(SIGMA(I,J),I=1,4),PA(J),LA(J),
      APB(J),LB(J),PC(J),LC(J),J=1,251)
103  FORMAT (5A6,4E10.4/I4,E9.4,I4,E9.4,I4,E9.4)
      READ (5,104) ((ISO(I,J),I=1,2),NE(J),(EGAM(I,J),FRAC(I,J),I=1,7),
      AJ=1,226)
104  FORMAT (A3,A4,I3,14F5.2)
C
C      READ IN MATERIAL DATA
C
9999 READ (5,100) (NAME(L),L=1,12)
      READ (5,105) OPT,JJ,(Z(J),J=1,JJ)
105  FORMAT (22I3)
      READ (5,106) WT,(WTF(J),J=1,JJ)
106  FORMAT (8F10.4)
      READ (5,107) (PHI(I),I=1,4),IN,T1,T2,LL,(T(L),L=1,LL)
107  FORMAT (4E10.4/I3,2E10.4,I3/(8E10.4))
      TN=FLOAT(IN)
      WRITE (6,200)
200  FORMAT (1H1,2X)
C
C      WRITE INPUT DATA--MATERIAL IDENTIFICATION INFORMATION, NEUTRON
C      FLUX, IRRADIATION TIME OR NUMBER OF CYCLES AND
C      IRRADIATION TIME PER CYCLE
C
      WRITE (6,201) (NAME(L),L=1,12)
201  FORMAT (1H ,29X,12A6///2X)
      IF (TN.NE.0.) GO TO 1
      WRITE (6,202) (PHI(I),I=1,4),T1
202  FORMAT (13H NEUTRON FLUX,4(5X,1PE10.3),20X,16HIRRADIATION TIME,
      A 1PE12.3,4H MIN//2X)
      GO TO 2
1  WRITE (6,203) (PHI(I),I=1,4),IN,T1
203  FORMAT (13H NEUTRON FLUX,4(5X,1PE10.3),10X,I4,7H CYCLES,5X,
      A16HIRRADIATION TIME,1PE10.3,4H MIN//2X)
C
C      CONVERT INPUT Z TO KEY TO REACTION INDEXING PARAMETERS
C
C      TRAP ELEMENTS REQUESTED, BUT NOT IN THE LIBRARY, WRITE MESSAGE
C
2  DO 1000 J=1,JJ

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      M=Z(J)
      IF (M.LT.9.OR.M.EQ.10.OR.M.EQ.39.OR.M.EQ.43.OR.M.EQ.61.OR.M.EQ.68.
ADR.M.EQ.76.OR.M.EQ.82.OR.M.GE.84.AND.M.LE.89.OR.M.GT.92) GO TO 11
      IF (M.NE.9) GO TO 3
      Z(J)=1
      GO TO 1000
3     IF (M.GT.39) GO TO 4
      Z(J)=Z(J)-9
      GO TO 1000
4     IF (M.GT.43) GO TO 5
      Z(J)=Z(J)-10
      GO TO 1000
5     IF (M.GT.61) GO TO 6
      Z(J)=Z(J)-11
      GO TO 1000
6     IF (M.GT.68) GO TO 7
      Z(J)=Z(J)-12
      GO TO 1000
7     IF (M.GT.76) GO TO 8
      Z(J)=Z(J)-13
      GO TO 1000
8     IF (M.GT.82) GO TO 9
      Z(J)=Z(J)-14
      GO TO 1000
9     IF (M.NE.83) GO TO 10
      Z(J)=68
      GO TO 1000
10    Z(J)=Z(J)-21
      GO TO 1000
11    WRITE (6,204) M
204   FORMAT (37H ELEMENT IS NOT LISTED IN LIBRARY  Z=,I3//2X)
      Z(J)=0
1000  CONTINUE
C
C     BEGIN CALCULATION OF END OF IRRADIATION DISINTEGRATION RATES
C
      N=0
      WRITE (6,205)
205   FORMAT(5X,7HELEMENT,11X,15HWEIGHT FRACTION13X,22HDPS AT ZERO DECAY
A TIME,17X,35HACTIVITY FRACTION PER NEUTRON GROUP/2X)
      DO 1012 J=1,JJ
      M=Z(J)
      IF (M.EQ.0) GO TO 1012
      KK=IE(M)
      KKK=IDEL(M)+KK-1
      PERC=ABS(WT*WTF(J))
C
C     WRITE ELEMENT NAME AND WEIGHT FRACTION
C
      WRITE (6,206) (ELEM(I,M),I=1,2),WTF(J)
206   FORMAT (2X/5X,2A6,10X,OPF8.5//2X)
      DO 1011 K=KK,KKK
      TOT=0.
      DO 1002 I=1,4
      FN(I)=PHI(I)*SIGMA(I,K)
      DFN(I)=FN(I)
1002  TOT=TOT+FN(I)
      IF (TN.NE.0.) GO TO 20
C
C     ACTIVATION WITHOUT CYCLING AND BURN-UP

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C      A=PERC*TOT
      AL=LA(K)
      ALX=EXP(-AL*T1)
      IPA=PA(K)

C
C      CALCULATE PARENT POST-IRRADIATION ACTIVITY (DIS/SEC)
C
      S1=A*(1.-ALX)
      IF (PB(K).EQ.0) GO TO 12
      BL=LB(K)
      BLX=EXP(-BL*T1)
      IPB=PB(K)

C
C      CALCULATE DAUGHTER POST-IRRADIATION ACTIVITY (DIS/SEC)
C
      S2=A*(BL*(1.-ALX)-AL*(1.-BLX))/(BL-AL)
      IF (PC(K).EQ.0) GO TO 13
      CL=LC(K)
      CLX=EXP(-CL*T1)
      IPC=PC(K)

C
C      CALCULATE GRAND DAUGHTER POST-IRRADIATION ACTIVITY (DIS/SEC)
C
      S3=A*((BL-AL)*(2.-CLX)-BL*CL*(ALX-CLX)/(CL-AL)+AL*CL*(BLX-CLX)
      A/(CL-BL))/(BL-AL)
      GO TO 14
12  IPB=0
      S2=0.
13  IPC=0
      S3=0.

C
C      CALCULATE NEUTRON GROUP ACTIVITY FRACTIONS
C
C      WRITE REACTION, POST-IRRADIATION DISINTEGRATION RATES, AND NEUTRON
C      GROUP ACTIVITY FRACTIONS
C
14  DO 1003 I=1,4
1003 DF(I)=DFN(I)/TOT
      WRITE (6,207) (REAC(I,K),I=1,5),S1,S2,S3,(DF(I),I=1,4)
207  FORMAT (10X,5A6,3(5X,1PE10.3),4(5X,OPF6.4))

C
C      CALCULATE DISINTEGRATION RATES FOR REQUIRED DECAY TIMES
C
      DO 1007 L=1,LL
      IF (T(L).GT.0.) GO TO 15
      AT=S1
      BT=S2
      CT=S3
      GO TO 16
15  ALT=EXP(-AL*T(L))
      AT=S1*ALT
      IF (IPB.EQ.0) GO TO 16
      BLT=EXP(-BL*T(L))
      BT=S2*BLT+BL*S1*(ALT-BLT)/(BL-AL)
      IF (IPC.EQ.0) GO TO 16
      CLT=EXP(-CL*T(L))
      CT=S3*CLT+CL*S2*(BLT-CLT)/(CL-BL)+CL*BL*S1*((ALT-CLT)/(CL-AL)
      A-(BLT-CLT)/(CL-BL))/(BL-AL)

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C      INCLUDE PARENT, DAUGHTER, GRAND DAUGHTER DISINTEGRATIONS RATES IN
C      PRODUCT ISOTOPE ARRAY--(1) IF ISOTOPE IS ALREADY PRESENT, ADD TO
C      TOTAL, (2) IF NEW ISOTOPE, INCLUDE IT IN ARRAY
C
16  IF (N.NE.0) GO TO 17
    PROD(1,1)=AT
    IP(1)=IPA
    N=1
    GO TO 18
17  DO 1004 NN=1,N
    IF (IPA.NE.IP(NN)) GO TO 1004
    PROD(NN,L)=PROD(NN,L)+AT
    GO TO 18
1004 CONTINUE
    N=N+1
    PROD(N,L)=AT
    IP(N)=IPA
18  IF (IPB.EQ.0) GO TO 1007
    DO 1005 NN=1,N
    IF (IPB.NE.IP(NN)) GO TO 1005
    PROD(NN,L)=PROD(NN,L)+BT
    GO TO 19
1005 CONTINUE
    N=N+1
    PROD(N,L)=BT
    IP(N)=IPB
19  IF (IPC.EQ.0) GO TO 1007
    DO 1006 NN=1,N
    IF (IPC.NE.IP(NN)) GO TO 1006
    PROD(NN,L)=PROD(NN,L)+CT
    GO TO 1007
1006 CONTINUE
    N=N+1
    PROD(N,L)=CT
    IP(N)=IPC
1007 CONTINUE
    GO TO 1011
C
C      ACTIVATION CALCULATION FOR CYCLING AND BURN-UP
C
20  AL=LA(K)
    ALX=EXP(-AL*T1)
    TOT=0.
    IPA=PA(K)
    S1=0.
    DO 1008 I=1,4
    SF(I)=60.*FN(I)/AD(M)
    SFX(I)=EXP(-SF(I)*T1)
    IF ((SFX(I)-ALX).NE.0.) GO TO 21
    ALC=TN
    DFN(I)=FN(I)*T1*ALC
    GO TO 1008
C
C      CALCULATE PARENT POST-IRRADIATION ACTIVITY (DIS/SEC)
C
21  ALC=(SFX(I)**TN-EXP(-TN*AL*(T1+T2)))/(SFX(I)-EXP(-AL*(T1+T2)))
    DFN(I)=FN(I)*(SFX(I)-ALX)/(AL-SF(I))*ALC
1008 TOT=TOT+DFN(I)
    S1=TOT*PERC*AL
    IF (PB(K).EQ.0) GO TO 22

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      IPB=PB(K)
      BL=LB(K)
      BLX=EXP(-BL*T1)
      S2=0.
C
C      CALCULATE DAUGHTER POST-IRRADIATION ACTIVITY (DIS/SEC)
C
      DO 1009 I=1,4
1009  S2=S2+FN(I)*(((SFX(I)-BLX)/(BL-SF(I))-(ALX-BLX)/(BL-AL))/(AL-SF(I))
      S2=S2*PERC*AL*BL*ALC
      IF (PC(K).EQ.0) GO TO 23
      IPC=PC(K)
      CL=LC(K)
      CLX=EXP(-CL*T1)
      S3=0.
C
C      CALCULATE GRAND DAUGHTER POST-IRRADIATION ACTIVITY (DIS/SEC)
C
      DO 1010 I=1,4
1010  S3=S3+FN(I)*(((SFX(I)-CLX)/(CL-SF(I))-(BLX-CLX)/(CL-BL))/(BL-SF(I)
      A)-((ALX-CLX)/(CL-AL)-(BLX-CLX)/(CL-BL))/(BL-AL))
      S3=S3*PERC*AL*BL*CL*ALC
      GO TO 14
22  IPB=0
      S2=0.
23  IPC=0
      S3=0.
      GO TO 14
C
C      GO BACK AND CALCULATE ACTIVITY AFTER DECAY TIMES AND INCLUDE IN
C      PRODUCT ISOTOPE ARRAY
C
1011 CONTINUE
1012 CONTINUE
      NEND=N
      DO 1015 L=1,LL
      WRITE (6,200)
      CTT=0.
      RTT=0.
      STT=0.
      IF (ABS(WT-1.).LT..00001) GO TO 24
      IF (WT.LT.0.) GO TO 25
C
C      WRITE THE DECAY TIME AND UNITS FOR CALCULATION
C
      WRITE (6,208) T(L)
208  FORMAT (23H TIME AFTER IRRADIATION,1PE12.4,4H MIN,20X,31HALL OUTPU
      AT PER CUBIC CENTIMETER//2X)
      GO TO 26
24  WRITE (6,209) T(L)
209  FORMAT (23H TIME AFTER IRRADIATION,1PE12.4,4H MIN,20X,19HALL OUTPU
      AT PER GRAM//2X)
      GO TO 26
25  WRITE (6,210) T(L)
210  FORMAT (23H TIME AFTER IRRADIATION,1PE12.4,4H MIN,20X,25HALL OUTPU
      AT FOR TOTAL MASS//2X)
C
C      CALCULATE DISINTEGRATION RATES (MCI), SOURCE STRENGTHS (MEV/SEC),
C      DOSE RATES (MR(C)/HR), AND TOTALS
C

```

```

26 DO 1014 N=1,NEND
   K=IP(N)
   NN=NE(K)
   IF (PROD(N,L).EQ.0.) GO TO 1014
   RT=0.
   ST=0.
   P=PROD(N,L)
   CT=P/3.7E7
   DO 1013 I=1,NN
     S(I)=FRAC(I,K)*EGAM(I,K)*P
     R(I)=S(I)*1.285E-8
     ST=ST+S(I)
1013 RT=RT+R(I)
C
C   WRITE PRODUCT ISOTOPE INVENTORY
C
   WRITE (6,211) (ISO(M,K),M=1,2),CT,RT,ST
211 FORMAT (5X,A3,A4,10X,1PE10.4,12H MILLICURIES,5X,1PE10.4,14H MR/HR
   AAT 1 M ,5X,1PE10.4,8H MEV/SEC/2X)
   IF (OPT.EQ.1) GO TO 27
   WRITE (6,212) (EGAM(I,K),I=1,NN)
212 FORMAT (12X,13HGAMMA ENERGY ,7(5X,0PF10.4))
   WRITE (6,213) (R(I),I=1,NN)
213 FORMAT (12X,13HMR/HR AT 1 M ,7(5X,1PE10.4))
   WRITE (6,214) (S(I),I=1,NN)
214 FORMAT (12X,13HMEV/SEC ,7(5X,1PE10.4)/2X)
27 CTT=CTT+CT
   RTT=RTT+RT
   STT=STT+ST
1014 CONTINUE
   WRITE (6,215) CTT,RTT,STT
215 FORMAT(2X/24H TOTAL MATERIAL ACTIVITY,10X,1PE10.4,12H MILLICURIES,
   A5X,1PE10.4,14H MR/HR AT 1 M ,5X,1PE10.4,8H MEV/SEC)
1015 CONTINUE
   GO TO 9999
   END

```

APPENDIX B

DATA LIBRARY LISTING

The NAC data library is listed below in three sections. The first section contains the target element names, atomic densities, and reaction indices. The second section contains the individual reactions and reaction parameters. The third section contains the product isotopes and gamma energy yields.

Section I contains the name of each element present in the data library, in order of increasing Z , on the first 12 cards. The next 8 cards contain one pair of reaction indices for each element. The first index identifies the first reaction for an element in the reaction table listed in Section II and the second index is the number of reactions for a given element. For example, the first two pair on the first card:

- 1 - first reaction for fluorine
- 3 - three fluorine reactions
- 4 - first reaction for sodium
- 4 - four sodium reactions

The final nine cards contain the atomic density (atoms per gram) of each element in order of increasing Z .

SECTION I ELEMENT NAMES, REACTION INDICES, ATOMIC DENSITIES (ATOMS/GRAMS)

FLUORINE	SODIUM	MAGNESIUM	ALUMINUM	SILICON	PHOSPHORUS
SULFUR	CHLORINE	ARGON	POTASSIUM	CALCIUM	SCANDIUM
TITANIUM	VANADIUM	CHROMIUM	MANGANESE	IRON	COBALT
NICKEL	COPPER	ZINC	GALLIUM	GERMANIUM	ARSENIC
SELENIUM	BROMINE	KRYPTON	RUBIDIUM	STRONTIUM	ZIRCONIUM
NIوبيUM	MOLYBDENUM	RUTHENIUM	RHODIUM	PALLADIUM	SILVER
CADMIUM	INDIUM	TIN	ANTIMONY	TELLURIUM	IODINE
XENON	CESIUM	BARIUM	LANTHANUM	CERIUM	PRASEODYMIUM
NEODYMIUM	SAMARIUM	EUROPIUM	GADOLINIUM	TERRIUM	DYSPROSIUM
HOLMIUM	THULIUM	YTTERBIUM	LUTTIUM	HAFNIUM	TANTALUM
TUNGSTEN	RHENIUM	IRIDIUM	PLATINUM	GOLD	MERCURY
THALLIUM	BISMUTH	THORIUM	PROTACTINIUM	URANIUM	
1 3	4 4	8 2	10 4	14 4	18 2
30 3	33 4	37 5	42 3	45 3	48 3
73 8	81 4	85 8	93 3	96 3	101 5
126 2	128 7	135 2	137 1	138 3	141 5
167 7	174 4	178 5	183 4	187 5	192 1
202 3	205 2	207 1	208 2	210 1	211 1
220 3	223 2	225 3	228 4	232 4	236 4
246 6					
3.164E22	2.615E22	2.477E22	2.233E22	2.145E22	1.945E22
1.508E22	1.540E22	1.503E22	1.340E22	1.258E22	1.181E22
1.079E22	1.022E22	1.026E22	9.471E21	9.215E21	8.644E21
7.630E21	7.539E21	7.184E21	7.050E21	6.885E21	6.604E21
5.926E21	5.856E21	5.641E21	5.585E21	5.359E21	5.247E21
4.720E21	4.747E21	4.579E21	4.533E21	4.386E21	4.341E21
4.181E21	4.000E21	3.964E21	3.841E21	3.782E21	3.703E21
3.481E21	3.626E21	3.375E21	3.329E21	3.273E21	3.235E21
3.058E21	3.003E21	2.945E21	2.878E21	2.591E21	2.570E21
					2.529E21

Section II consists of two cards for each of the 251 reactions. The first card contains the reaction, followed by the activation cross sections (cm^2/g) for the four neutron groups in order of decreasing energy. Blanks indicate that the particular cross section is zero or unknown. The second card contains up to three pairs of numbers, each pair corresponding to a radioactive isotope. The first number of each pair is an index which identifies the product isotope (Section III). The second number of the pair is the isotope decay constant (min^{-1}).

For example for the first three reactions:

product isotope 3 is F^{20}

product isotope 2 is O^{19}

product isotope 1 is N^{16}

SECTION II REACTION, CROSS SECTIONS (CM²/GRAM), PRODUCT ISOTOPE INDEX,
DECAY CONSTANT (MIN⁻¹)

F 19 (N,G) F 20	2.286-6	9.130-5	3.393-5	2.814-4
3 3.707+0				
F 19 (N,P) O 19	7.581-5			
2 1.435+0				
F 19 (N,A) N 16	3.786-4			
1 5.635+0				
NA 23 (N,G) NA 24	1.602-6	2.689-5	4.851-4	1.124-3
6 7.702-4				
NA 23 (N,P) NF 23	3.927-5			
4 1.019+0				
NA 23 (N,A) F 20	1.749-5			
3 3.707+0				
NA 23 (N,2N) NA 22	1.649-7			
5 5.059-7				
MG 24 (N,P) NA 24	4.279-3			
6 7.702-4				
MG 26 (N,G) MG 27	1.678-8	2.853-7	2.348-6	5.424-5
7 7.374-2				
AL 27 (N,G) AL 28	8.262-6	6.394-4	1.822-4	4.212-3
9 3.000-1				
AL 27 (N,P) MG 27	1.200-4			
7 7.374-2				
AL 27 (N,A) MA 24	2.552-5			
6 7.702-4				
AL 27 (N,2N) AL 26	5.192-4			
8 1.635-12				
SI 28 (N,P) AL 28	2.295-4			
9 3.000-1				
SI 29 (N,P) AL 29	2.703-6			
10 1.035-1				
SI 30 (N,G) SI 31	3.288-9	5.656-8	2.499-6	5.469-4
11 4.077-3				
SI 30 (N,A) MG 27	8.042-6			
7 7.374-2				
P 31 (N,P) SI 31	7.455-4			
11 4.077-3				
P 31 (N,A) AL 28	6.866-5			
9 3.000-1				
S 34 (N,A) SI 31	2.409-6			
11 4.077-3				
S 36 (N,S) S 37	2.549-11	3.527-10	1.484-8	3.598-7
12 1.375-1				
CL 35 (N,2N) CL 34M	9.585-6			
13 2.139-2				
CL 37 (N,G) CL 38	1.370-7	1.416-6	6.153-5	1.443-3
14 1.858-2				
CL 37 (N,P) S 37	2.924-8			
12 1.375-1				
A 40 (N,G) A 41	6.974-7	7.220-6	3.182-4	7.358-3
15 6.301-3				
K 39 (N,G) K 40	2.119-7	2.194-6	9.671-5	2.236-3
17 1.039-13				
K 39 (N,2N) K 38	3.830-7			

16 9.002-2				
K 41 (N,G) K 42	1.052-7	1.090-6	4.804-5	1.110-4
18 9.242-4				
K 41 (N,A) CL 38	8.821-6			
14 1.858-2				
CA 42 (N,P) K 42	5.679-5			
18 9.242-4				
CA 46 (N,G) CA 47--SC 47	7.832-12	8.720-11	3.845-9	1.331-7
19 1.024-4 24 1.416-4				
CA 48 (N,G) CA 49	2.331-9	2.406-8	9.752-7	2.457-5
20 7.877-3				
SC 45 (N,G) SC 46	2.259-5	2.353-4	1.038-2	2.399-2
23 5.729-6				
SC 45 (N,A) K 42	4.958-6			
18 9.242-4				
SC 45 (N,2N) SC 44M--SC 44	4.336-7			
21 2.003-4 22 2.962-3				
SC 45 (N,2N) SC 44	5.577-7			
22 2.962-3				
TI 46 (N,P) SC 46	1.589-5			
23 5.729-6				
TI 47 (N,P) SC 47	3.400-5			
24 1.416-4				
TI 48 (N,P) SC 48	4.312-7			
25 2.626-4				
TI 50 (N,G) TI 51	5.344-9	7.403-8	3.265-6	7.551-5
26 1.197-2				
TI 50 (N,A) CA 47--SC 47	1.344-10			
19 1.024-4 24 1.416-4				
V 51 (N,G) V 52	1.379-5	3.966-4	1.843-3	4.266-2
27 1.844-1				
V 51 (N,P) TI 51	3.560-7			
26 1.197-2				
V 51 (N,A) SC 48	1.957-7			
25 2.626-4				
CR 50 (N,G) CR 51	8.222-7	8.500-6	3.747-4	8.670-3
29 1.733-5				
CR 50 (N,2N) CR 49	3.646-9			
28 1.650-2				
CR 52 (N,P) V 52	2.772-5			
27 1.844-1				
MN 55 (N,G) MN 56	2.505-5	3.740-4	9.454-3	1.199-1
31 4.501-3				
MN 55 (N,A) V 52	2.128-5			
27 1.844-1				
MN 55 (N,2N) MN 54	6.554-6			
30 1.534-6				
FE 54 (N,P) MN 54	8.216-5			
30 1.534-6				
FE 54 (N,A) CR 51	2.332-7			
29 1.733-5				
FE 56 (N,P) MN 56	1.698-5			
31 4.501-3				
FE 58 (N,G) FE 59	2.567-9	2.661-8	1.174-6	5.370-4
32 1.066-5				
CO 59 (N,G) CO 60M--CO 60	1.244-5	2.116-4	6.346-3	1.313-1
36 6.602-2 37 2.502-7				
CO 59 (N,G) CO 60	1.346-5	1.222-4	8.015-3	1.641-1
37 2.502-7				
CO 59 (N,P) FE 59	3.443-4			

32	1.066-5				
CO 59	(N,A) MN 56	1.162-6			
31	4.501-3				
CO 59	(N,2N) CO 58M--CO 58	9.577-7			
34	1.284-3				
35	6.665-6				
CO 59	(N,2N) CO 58	9.577-7			
35	6.665-6				
NI 58	(N,P) CO 58M--CO 58	9.038-5			
34	1.284-3				
35	6.665-6				
NI 58	(N,P) CO 58	9.119-4			
35	6.665-6				
NI 58	(N,NP) CO 57	2.973-4			
33	1.783-6				
NI 58	(N,2N) NI 57	4.358-8			
38	3.067-4				
NI 60	(N,P) CO 60	1.342-5			
37	2.502-7				
NI 62	(N,A) FE 59	5.257-8			
32	1.066-5				
NI 64	(N,G) NI 65	1.386-8	1.436-7	6.333-6	1.464-4
39	4.444-3				
CU 63	(N,G) CU 64	6.810-6	1.688-4	3.418-4	4.717-3
41	8.887-4				
CU 63	(N,A) CU 60	4.718-6			
37	2.502-7				
CU 63	(N,2N) CU 62	1.501-6			
40	6.974-2				
CU 65	(N,G) CU 66	1.180-5	5.629-5	5.602-4	4.355-3
42	1.359-1				
CU 65	(N,2N) CU 64	1.189-6			
41	8.887-4				
ZN 64	(N,G) ZN 65	1.596-7	1.668-6	7.347-5	1.700-3
45	1.969-6				
ZN 64	(N,P) CU 64	3.156-5			
41	8.887-4				
ZN 64	(N,2N) ZN 63	1.740-7			
44	1.926-2				
ZN 66	(N,P) CU 66	8.372-6			
42	1.359-1				
ZN 67	(N,P) CU 67	1.014-7			
43	1.904-4				
ZN 68	(N,G) ZN 69M	5.394-9	1.332-7	5.881-6	2.083-4
46	8.252-4				
ZN 68	(N,A) NI 65	3.421-8			
39	4.444-3				
ZN 70	(N,G) ZN 71	1.569-10	3.889-9	1.714-7	3.963-6
47	3.151-1				
GA 69	(N,G) GA 70	1.087-5	6.186-5	2.529-4	5.848-3
49	3.285-2				
GA 69	(N,P) ZN 69M	1.166-6			
46	8.252-4				
GA 69	(N,2N) GA 68	3.107-6			
48	1.019-2				
GA 71	(N,G) GA 72		6.420-5	5.972-4	1.381-2
50	8.622-4				
GE 70	(N,P) GA 70	5.924-5			
49	3.285-2				
GE 70	(N,2N) GE 69	4.251-7			
52	2.888-5				
GE 72	(N,P) GA 72	2.271-8			

50	8.622-4							
GE 73	(M, P) GA 73	2.381-5						
51	2.407-3							
GE 74	(M, A) 7M 71	1.222-5						
47	3.157-1							
GE 76	(M, G) GE 77--AS 77	8.357-10	4.091-8	1.768-6	4.088-5			
55	1.050-3	58 2.962-4						
GE 76	(M, G) GE 77M--AS 77	7.187-10	3.518-8	1.520-6	3.516-5			
53	7.702-2	58 2.962-4						
GE 76	(M, G) GE 77M--GE 77--AS 77	1.170-10	4.730-9	2.480-7	4.620-6			
54	7.702-2	55 1.050-3	58 2.962-4					
AS 75	(M, G) AS 76	1.809-5	1.616-4	1.199-3	2.776-2			
57	4.360-4							
AS 75	(M, A) GA 72	1.733-7						
50	8.622-4							
AS 75	(M, 2M) AS 74	1.816-6						
56	2.676-5							
SE 74	(M, G) SE 75	1.195-6	3.660-6	5.995-5	1.386-3			
60	3.084-6							
SE 77	(M, P) AS 77	6.955-6						
58	2.985-4							
SE 80	(M, G) SE 81M		8.086-8	3.961-6	9.153-5			
61	1.220-2							
SE 82	(M, G) SE 83M--SE 83--RR 83			3.841-5	1.738-3			
62	6.028-1	63 2.773-2	67 5.023-3					
SE 82	(M, G) SE 83--RR 83			1.384-4	3.043-3			
63	2.773-2	67 5.023-3						
RR 79	(M, G) RR 80		2.640-4	1.124-3	2.599-2			
65	3.939-2							
RR 79	(M, A) AS 76	4.882-7						
57	4.360-4							
RR 81	(M, G) RR 82	6.675-6	1.568-3	8.602-3	9.850-3			
66	3.209-4							
RR 81	(M, A) AS 78	1.027-4						
59	7.617-3							
RR 81	(M, 2M) RR 80M--RR 80	1.526-6						
64	5.767-3	65 5.939-2						
RR 78	(M, G) RR 79	3.831-9	3.962-8	1.756-6	4.040-5			
68	5.398-4							
RR 80	(M, G) RR 81	7.056-8	2.300-5	6.778-4	1.246-2			
69	6.413-12							
RR 84	(M, G) RR 85M	2.231-6	3.301-4	4.948-4	6.244-5			
70	2.525-3							
RR 84	(M, G) RR 85M--RR 85	9.512-6	1.407-3	2.109-3	2.662-4			
70	2.626-3	71 1.238-7						
RR 84	(M, G) RR 85	7.019-6	1.046-3	1.558-3	1.972-4			
71	1.238-7							
RR 86	(M, G) RR 87		5.903-8	2.603-6	6.015-5			
72	8.887-3							
RR 85	(M, G) RR 86	1.529-7	9.959-5	2.198-4	3.717-3			
74	2.530-5							
RR 85	(M, 2M) RR 84	3.685-6						
73	1.459-5							
RR 87	(M, G) RR 88	3.659-6	2.550-5	1.285-5	1.892-4			
75	3.894-2							
RR 87	(M, 2M) RR 86	1.703-6						
74	2.530-5							
SR 84	(M, G) SR 85M	4.124-10	4.246-9	1.872-7	5.952-6			
77	9.003-3							
SR 84	(M, G) SR 85M--SR 85	3.081-9	3.819-8	1.685-6	6.033-5			

76	9.903-3	78	7.618-5				
SR 84	(N,G) SR 85			3.493-9	4.243-8	1.872-6	6.628-5
78	7.618-5						
SR 86	(N,G) SR 87M			8.522-8	8.799-7	3.885-5	8.982-4
79	4.120-3						
ZR 90	(N,2N) ZR 89			7.668-7			
84	1.453-4						
ZR 94	(N,G) ZR 95--NR95M--NR95				4.674-7	5.934-9	1.397-6
85	7.382-6	90	1.283-4	1.372-5			
ZR 94	(N,G) ZR 95--NR 95				2.291-5	2.909-7	6.848-5
85	7.382-6	91	1.372-5				
ZR 94	(N,A) SR 91--Y 91M--Y 91			2.321-10			
80	1.195-3	82	1.378-2	83	8.242-6		
ZR 94	(N,A) SR 91--Y 91			1.613-10			
81	1.195-3	83	8.242-6				
ZR 96	(N,G) ZR 97--NR 97			7.836-8	6.549-6	2.935-7	1.446-5
86	6.796-4	92	9.614-3				
NR 93	(N,G) NR 94M--NR 94			1.545-5	3.927-3	4.655-3	5.992-3
88	1.050-1	89	6.301-11				
NR 93	(N,2N) NR 92			3.977-8			
87	4.813-5						
MD 92	(N,G) MD 93M			5.400-8	2.298-5	1.516-6	4.000-5
93	1.650-3						
MD 92	(N,P) NR 92			1.319-6			
87	4.814-5						
MD 92	(N,A) ZR 89			1.695-8			
84	1.462-4						
MD 95	(N,P) NR 95			9.858-8			
91	1.375-5						
MD 97	(N,P) NR 97			1.744-5			
92	9.367-3						
MD 98	(N,G) MD 99			6.686-6	2.853-4	6.334-3	6.110-4
94	1.724-4						
MD100	(N,G) MD101--FC101			5.118-8	2.233-5	3.367-5	9.706-5
95	4.621-2	96	4.951-2				
RD102	(N,G) RD103			3.400-6	6.265-4	3.443-3	2.192-3
97	1.222-5						
RD104	(N,G) RD105--RH105			4.390-6	2.794-4	1.813-3	6.136-4
97	2.606-3	100	3.224-4				
RH104	(N,G) RH105M--RH105			3.306-6	4.075-4	4.224-3	8.950-2
99	1.575-1	100	3.224-4				
PD102	(N,G) PD103			1.722-8	6.760-6	1.837-5	1.742-4
101	2.876-5						
PD108	(N,G) PD109M			2.244-7	7.878-5	4.777-4	2.181-3
103	1.459-1						
PD110	(N,G) PD111--AG111M--AG111			2.438-7	1.150-4	3.036-3	1.285-4
104	3.150-2	107	5.776-1	108	6.418-5		
AG107	(N,G) AG108			1.534-5	2.342-3	1.457-2	7.142-2
105	2.888-1						
AG107	(N,2N) AG106M			2.421-8			
104	5.728-5						
AG109	(N,G) AG110M			3.597-7	5.848-5	3.801-5	4.746-3
106	1.904-6						
AG109	(N,P) PD109M			3.084-8			
103	1.459-1						
AG109	(N,2N) AG108			4.537-4			
105	2.888-1						
CD106	(N,G) CD107			2.040-8	9.100-6	1.933-5	5.252-5
109	1.724-3						
CD110	(N,G) CD111M			2.075-7	1.006-4	5.933-4	1.067-4

110	1.386-2				
CD114 (N,G)	CD115M-CD115-IN115M	4.357-7	2.089-4	1.133-3	1.378-3
111	1.130-5 112 2.179-4 117	2.567-3			
CD114 (N,G)	CD115--IN115M	5.458-8	2.662-5	1.495-4	1.752-4
112	2.179-4 117 2.567-3				
CD116 (N,G)	CD117M-CD117-IN117M	1.269-7	6.186-5	1.451-4	4.895-4
113	3.983-3 114 1.386-2 119	6.080-3			
CD116 (N,2N)	CD115M-CD115-IN115M	3.410-8			
111	1.130-5 112 2.179-4 117	2.567-3			
CD116 (N,2N)	CD115--IN115M	3.399-8			
112	2.179-4 117 2.567-3				
IN113 (N,G)	IN114M--IN114	2.610-7	4.196-5	2.478-3	9.988-3
115	9.627-6 116 5.776-1				
IN113 (N,G)	IN114	9.329-9	1.496-6	8.888-5	3.569-4
116	5.776-1				
IN115 (N,G)	IN116M	1.355-5	1.199-3	2.432-1	6.256-1
118	1.286-2				
IN115 (N,P)	CD115--IN115M	2.014-5			
112	2.179-4 117 2.567-3				
IN115 (N,2N)	IN114M--IN114	1.130-6			
115	9.627-6 116 5.776-1				
SM112 (N,G)	SM113	2.885-8	1.220-5	3.500-5	5.034-5
120	4.077-6				
SM122 (N,G)	SM123	9.499-8	4.304-5	8.457-5	3.064-5
121	1.754-2				
SM124 (N,G)	SM125--SM125	1.205-7	5.248-5	4.757-5	4.875-5
122	7.146-2 128 4.881-7				
SB121 (N,G)	SB122M	1.690-7	4.857-5	2.761-4	4.320-4
124	1.980-1				
SB121 (N,G)	SB122	4.900-6	1.410-3	8.676-3	1.252-2
125	1.724-4				
SB121 (N,2N)	SB120	2.041-6			
123	8.301-5				
SB 123 (N,G)	SB124M--SB124	2.001-8	7.850-6	6.233-5	1.017-5
126	3.300-2 127 8.023-6				
SB123 (N,G)	SB124	8.334-7	3.284-4	2.110-3	4.249-3
127	8.023-6				
SB123 (N,2N)	SB122	2.020-6			
125	1.724-4				
TF122 (N,G)	TF123M	4.832-8	1.724-5	9.843-5	1.026-4
129	4.621-6				
TF124 (N,G)	TF125M	7.835-8	3.496-5	1.405-4	1.188-3
130	8.291-6				
TF126 (N,G)	TF127M--TF127	2.105-8	1.104-5	5.325-5	6.389-5
131	4.560-6 132 1.229-3				
TF126 (N,G)	TF127		9.744-5	4.634-4	5.679-4
132	1.229-3				
TF128 (N,G)	TF129M--TF129	1.113-8	5.406-6	8.313-6	1.809-5
133	1.429-5 134 1.034-2				
TF128 (N,G)	TF129	9.982-7	4.938-5	7.198-5	1.568-4
134	1.034-2				
TF130 (N,G)	TF131--I 131	4.552-7	1.065-4	1.617-4	2.886-4
135	2.795-2 140 5.924-5				
I 127 (N,G)	I 128	5.530-6	1.586-3	6.754-3	2.050-2
138	2.783-2				
I 127 (N,P)	TF127	7.067-9			
132	1.229-3				
I 127 (N,A)	SB124	5.099-10			
127	8.023-6				
I 127 (N,2N)	I 126	4.476-6			

137	3.629-5				
XF124	(N,G) XE125--I 125	6.232-7	4.313-5	2.618-4	
141	6.418-4 136 8.013-6				
XE128	(N,G) XE129M	1.213-8	5.825-6	3.378-5	1.487-4
142	5.332-5				
XE132	(N,G) XE133	3.333-7	1.055-4	2.507-4	2.081-4
143	9.145-5				
XE134	(N,G) XE135		1.084-5	8.916-6	8.079-5
144	1.269-3				
XE136	(N,G) XE137--CS137		6.636-6	2.017-6	4.904-5
145	1.777-1 149 4.951-8				
CS133	(N,G) CS134	5.815-6	1.332-3	2.113-2	9.470-2
147	5.975-7				
CS133	(N,P) XE133	6.146-8			
143	9.145-5				
CS133	(N,A) I 130	7.110-9			
139	9.242-4				
CS133	(N,2N) CS132	4.755-6			
146	7.429-5				
HA132	(N,G) RA133M--HA133	1.021-8	1.910-8	5.538-6	1.702-5
151	1.758-7 152 2.962-4				
HA132	(N,P) CS132	2.237-8			
146	7.764-5				
HA134	(N,G) HA135M	4.494-7	1.767-5	4.911-4	1.709-4
153	4.030-4				
HA136	(N,P) CS136	5.263-10			
148	3.703-5				
HA138	(N,P) CS138	3.360-6			
150	2.153-2				
LA139	(N,G) LA140	8.950-7	2.533-4	8.192-3	3.099-2
154	2.876-4				
CE138	(N,2N) CE137M--CE137	1.300-8			
155	3.398-4 156 1.327-3				
CE140	(N,G) CE141		1.112-5	3.892-5	9.476-4
157	1.459-5				
CE142	(N,G) CE143		9.522-6	2.025-5	3.593-4
158	3.501-4				
PR141	(N,G) PR142	8.822-7	2.448-4	2.149-3	3.884-2
159	5.874-4				
ND146	(N,G) ND147--PM147	1.671-8	4.037-6	4.913-4	6.523-4
160	4.359-5 163 5.023-7				
ND148	(N,G) ND149--PM149	1.328-7	2.396-5	4.913-4	6.523-4
161	6.080-3 164 2.179-4				
ND150	(N,G) ND151--PM151	6.499-8	1.174-5	2.362-5	2.817-4
162	5.776-2 165 4.126-4				
SM152	(N,G) SM153	6.144-7	1.826-4	1.603-2	1.200-1
166	2.458-4				
SM154	(N,G) SM155	2.734-7	5.343-5	4.784-4	3.624-3
167	3.150-2				
EU151	(N,G) EU152M	1.968-4	1.292-2	5.228-1	9.541+0
168	1.064-7				
EU151	(N,G) EU152	4.380-5	2.874-3	1.157-1	2.121+0
169	1.242-3				
EU153	(N,G) EU154		3.809-3	2.347-1	7.691-1
170	8.242-8				
GD158	(N,G) GD159	8.172-7	1.402-4	2.636-3	3.062-3
171	6.418-4				
GD160	(N,G) GD161	7.196-7	1.204-4	2.357-3	5.069-4
172	1.873-1				
TR159	(N,G) TR160	2.208-5	4.720-3	6.125-2	1.401-1

173 6.665-6				
DY164 (N,G) DY165M--DY165	3.320-5	1.623-3	6.807-2	1.679+0
174 5.545-1 175 4.916-3				
DY164 (N,G) DY165	1.328-5	6.497-4	2.968-2	6.715-1
175 4.916-3				
H0165 (N,G) H0166M	1.006-5	2.316-3	5.114-2	1.907-1
176 1.100-6				
TM169 (N,G) TM170	1.435-5	2.364-3	4.290-2	3.638-1
177 3.726-6				
YB168 (N,G) YB169	1.655-6	4.160-5	1.861-3	4.294-2
178 1.554-5				
LU175 (N,G) LU176M	2.111-5	4.153-3	1.857-1	9.441-2
179 3.122-3				
LU176 (N,G) LU177	5.698-6	4.539-4	2.290-2	2.708-1
180 7.183-5				
HF174 (N,G) HF175	1.447-7	7.097-6	3.483-4	7.316-3
181 6.863-6				
HF179 (N,G) HF180M	4.650-7	5.412-6	1.796-2	8.438-3
182 2.100-3				
HF180 (N,G) HF181	7.594-6	1.994-4	3.128-3	1.344-2
183 1.119-5				
TA181 (N,G) TA182	1.216-5	2.454-3	8.288-2	5.618-2
185 4.077-6				
TA181 (N,2N) TA180M	4.497-6			
184 1.417-3				
W 184 (N,G) W 185	7.966-7	2.065-4	1.332-3	1.612-3
187 6.245-6				
W 186 (N,G) W187	4.583-6	1.406-4	1.691-3	2.617-2
188 4.813-4				
W 186 (N,P) TA186	6.602-10			
186 6.601-2				
RE185 (N,G) RE186	5.232-6	1.854-3	4.079-2	1.002-1
189 5.291-6				
RE187 (N,G) RE188	4.204-6	9.999-4	1.770-2	1.079-1
190 6.932-4				
IR191 (N,G) IR192M--IR192	7.312-6	6.398-4	1.694-2	2.483-1
191 4.881-1 192 6.478-6				
IR191 (N,G) IR192	1.968-5	1.724-3	4.565-2	6.555-1
192 6.478-6				
IR193 (N,G) IR194	6.633-4	1.448-1	3.297+1	2.014-1
193 6.080-4				
PT192 (N,G) PT193M	2.614-9	6.883-7	1.053-5	1.586-4
194 1.118-4				
PT196 (N,G) PT197	7.844-5	4.025-7	1.118-4	6.176-4
195 5.415-4				
PT198 (N,G) PT199--AU199	3.059-7	7.279-5	6.884-4	7.140-4
196 2.311-2 199 1.537-4				
PT198 (N,2N) PT197	1.646-3			
195 5.415-4				
AU197 (N,G) AU198	5.731-6	1.210-3	1.689-2	2.427-1
198 1.782-4				
AU197 (N,P) PT197	3.293-6			
195 5.415-4				
AU197 (N,A) IR194	6.431-11			
193 6.080-4				
AU197 (N,2N) AU196	1.824-5			
197 7.788-5				
HG196 (N,G) HG197M--HG197	4.384-7	8.719-6	7.423-5	1.802-3
201 4.814-4 202 1.777-4				
HG198 (N,G) HG199M	3.824-7	9.261-5	2.081-4	4.545-6

203	1.650-2				
HG200 (N,P)	AU200	6.682-7			
200	1.444-2				
HG202 (N,G)	HG203	8.949-5	2.181-6	2.008-5	3.393-3
204	1.024-5				
TL203 (N,P)	HG203	1.739-9			
204	1.024-5				
TL203 (N,2N)	TL202	1.112-6			
205	4.006-5				
RI209 (N,G)	RI210--PII210		3.047-6	2.088-4	4.401-5
206	9.627-5	207 3.483-6			
TH232 (N,G)	TH233--PA233--U233	4.535-6	8.604-4	4.838-3	1.564-2
213	3.136-2	216 1.777-5	218 1.772-8		
TH232 (N,2N)	TH231--PA231	5.904-5			
211	4.501-4	214 3.916-11			
PA231 (N,G)	PA232--U232--TH228	2.929-6	9.335-4	1.283-1	4.190-1
215	3.667-4	217 1.772-8	209 6.863-7		
U 234 (N,G)	U235--TH231--PA231	5.726-9	5.179-8	3.755-7	8.775-6
220	1.904-15	211 4.501-4	214 3.916-11		
U 234 (N,2N)	U233--TH229--RA225	4.700-11			
218	8.069-12	210 1.795-9	208 3.239-5		
U 235 (N,G)	U 236--TH232	9.998-7	1.645-5	3.549-4	1.451-3
221	4.847-14	212 9.431-17			
U 235 (N,2N)	U 234	2.664-7			
219	5.291-11				
U 238 (N,G)	U239--NP239--PII239	1.375-4	8.644-4	6.482-3	5.449-3
223	2.949-2	225 2.038-4	226 5.415-11		
U 238 (N,2N)	U237--NP237--PA233	5.404-5			
222	7.131-5	224 5.975-13	216 1.772-8		

Section III consists of one card for each of the 226 radioactive product isotopes. Each card contains the isotope name followed by the number of gamma rays emitted. The remainder of the card contains up to seven pairs of numbers. Each pair consists of a gamma ray energy (MeV) followed by the fraction of gamma rays emitted at that energy per disintegration. A few isotopes emit gammas at more than seven energies. In these cases, the lowest energy groups are averaged into a single gamma energy.

SECTION III PRODUCT ISOTOPE NAME, NUMBER OF GAMMAS, GAMMA ENERGY (MEV),
FRACTION OF GAMMAS AT A GIVEN ENERGY

N 16	3	7.1	.08	6.13	.99	2.15	.01												
O 19	4	1.44	.03	1.37	.56	.2	.97	.112	.03										
F 20	1	1.63	1.																
NE 23	2	1.65	.03	.44	.97														
NA 22	1	1.28	1.0																
NA 24	2	2.75	1.	1.37	1.0														
MG 27	3	1.02	.43	.834	1.	1.75	.01												
AL 26	3	2.96	.003	1.83	.997	1.12	.04												
AL 28	1	1.78	1.																
AL 29	2	2.43	.062	1.28	.938														
SI 31	1	1.27	.001																
S 37	1	3.09	.9																
CL 34M	5	.145	1.	4.1	.009	3.3	.24	2.13	.751	1.16	.24								
CL 38	2	2.16	.47	1.6	.31														
A 41	1	.29	.01																
K 38	1	2.16	1.																
K 40	1	1.46	.1																
K 42	2	1.53	.18	.32	.01														
CA 47	3	1.3	.93	.81	.07	.5	.07												
CA 49	3	4.68	.01	4.05	.1	3.1	.89												
SC 44M	1	.27	1.																
SC 44	4	2.69	.002	2.28	.002	1.5	.01	1.14	.03										
SC 46	2	1.12	1.	.89	1.														
SC 47	1	.16	.7																
SC 48	3	1.31	1.	1.04	1.	.99	1.												
TI 51	3	.93	.048	.61	.014	.322	.952												
V 52	1	1.44	1.																
CR 49	3	.15	.14	.09	.35	.06	.15												
CR 51	3	.65	.01	.33	.01	.32	.09												
MM 54	1	.835	1.																
MM 56	7	3.39	.002	2.96	.004	2.66	.007	2.52	.012	2.12	.153	1.81	.296	.845	1.				
FE 59	3	1.29	.43	1.1	.57	.2	.03												
CO 57	2	.707	.002	.136	.998														
CO 58M	1	.025	1.																
CO 58	3	1.65	.005	.81	.016	.805	.995												
CO 60M	1	.059	1.																
CO 60	2	1.33	1.	1.17	1.														
MI 57	3	1.9	.14	1.37	.86	.127	.14												

GE 77M	1	.159	1.											
GE 77	7	2.32	.01	1.96	.04	1.5	.07	1.08	.23	.709	.4	.416	.82	.153 1.
AS 74	7	2.2	.003	1.83	.003	1.61	.003	1.2	.003	1.	.003	.635	.006	.596 .154
AS 76	5	2.06	.02	1.41	.01	1.21	.12	.648	.03	.561	.37			
AS 77	4	.525	.008	.245	.025	.16	.003	.086	.001					
AS 78	7	2.65	.01	2.24	.01	1.82	.07	1.31	.19	.83	.08	.695	.54	.27 .06
SE 75	7	.77	.02	.628	.01	.427	.02	.273	1.4	.264	1.	.199	.08	.129 1.33
SE 81M	1	.103	1.											
SE 83M	4	2.02	.35	1.01	.88	.65	.18	.35	.11					
SE 83	7	2.29	.1	1.88	.1	1.31	.1	1.06	.1	.833	.1	.524	.1	.358 .1
RR 80M	2	.049	1.	.037	1.									
RR 80	1	.62	.13											
RR 82	7	1.32	.09	1.04	.1	.828	.09	.777	.27	.698	.1	.619	.14	.554 .31
RR 83	2	.087	.2	.046	1.									
KR 79	7	.3	.05	.217	.2	.201	.2	.181	.2	.136	.2	.084	.2	.045 .2
KR 81	1	.012	1.											
KR 85M	1	.15	1.											
KR 85	1	.514	.007											
KR 87	4	2.57	.22	2.05	.03	.847	.13	.403	.65					
RB 84	3	1.9	.008	1.01	.005	.88	.74							
RB 86	1	1.08	.09											
RB 88	7	4.87	.003	3.24	.02	2.68	.03	2.11	.01	1.84	.23	1.39	.01	.899 .15
SR 85M	2	.232	.012	.225	.988									
SR 85M	1	.15	1.											
SR 85	1	.513	1.											
SR 87M	1	.388	1.											
SR 91	2	1.03	.508	.748	.492									
SR 91	2	1.41	.171	.93	.073									
Y 91M	1	.551	1.											
Y 91	1	1.19	.002											
ZR 89	1	.913	1.											
ZR 95	2	.76	.439	.726	.561									
ZR 97	4	1.72	.1	1.15	.6	1.02	.3	.747	1.					
NR 92	3	1.83	.02	.93	.01	.9	.98							
NR 94M	1	.042	1.											
NR 94	2	.874	1.	.7	1.									
NR 95M	1	.231	1.											
NR 95	1	.765	1.											
NR 97	2	1.02	.01	.665	.99									
NO 93M	3	1.48	1.	.685	1.	.264	1.							
NO 99	4	.78	.04	.74	.1	.372	.01	.181	.04					
MO101	7	2.08	.16	1.56	.11	1.38	.19	1.02	.41	.704	.2	.591	.46	.191 .27
TC101	5	.72	.01	.635	.01	.545	.08	.385	.02	.307	.91			
RU103	7	.61	.065	.555	.005	.498	.885	.44	.005	.362	.003	.297	.004	.053 .015
RU105	7	1.35	.001	.966	.03	.874	.043	.726	.48	.67	.15	.475	.26	.318 .14
RH105M	1	.13	1.											
RH105	1	.319	1.											
PD103	4	.538	.001	.36	.001	.095	.1	.04	.9					
PD109M	1	.19	1.											
PD111	4	1.4	.08	.81	.01	.6	.13	.377	.05					
AG106M	7	1.55	.296	1.05	.6	.82	.48	.72	.49	.612	.23	.513	.86	.41 .67
AG108	3	.633	.019	.617	.01	.433	.03							
AG110M	7	1.5	.4	.937	.34	.885	.72	.764	.3	.706	.37	.656	.93	.619 .1
AG111M	1	.065	1.											
AG111	3	.342	.061	.247	.012	.095	.001							
CD107	2	.846	.004	.093	1.									
CD111M	2	.247	.94	.15	.29									
CD115M	3	1.3	.01	.935	.023	.485	.003							
CD115	6	.523	.24	.49	.122	.763	.002	.26	.017	.23	.006	.033	.002	
CD117M	2	1.27	1.	.28	1.									

GD117	1	.425	1.																	
IN114M	3	.722	.04	.556	.04	.192	.96													
IN114	1	1.3	.002																	
IN115M	1	.335	1.																	
IN116M	7	2.09	.19	1.49	.1	1.27	.83	1.09	.57	.82	.15	.406	.4	.137	.06					
IN117M	3	.822	.04	.315	.04	.161	.23													
SN113	2	.393	1.	.255	0.02															
SN123	1	.153	1.																	
SN125	4	1.39	.019	1.07	.003	.64	.003	.326	.997											
SB120	4	1.18	1.	1.04	1.	.199	1.	.089	.9											
SB122M	2	.075	1.	.061	1.															
SB122	4	1.26	.007	1.14	.008	.69	.031	.564	.679											
SB124M	1	.018	1.																	
SB124	7	2.09	.07	1.69	.52	1.37	.04	1.32	.03	1.05	.04	.722	.14	.645	.99					
SB125	7	.668	.02	.633	.11	.598	.24	.462	.41	.176	.07	.11	.21	.035	.76					
TE123M	2	.159	1.	.089	1.															
TE125M	2	.11	1.	.035	1.															
TE127M	1	.089	.98																	
TE127	5	.418	.008	.36	.001	.203	.001	.145	.001	.059	.005									
TE129M	1	.106	1.																	
TE129	4	1.12	.1	1.08	.007	.44	.09	.027	.71											
TE131	5	1.13	.1	.92	.05	.6	.05	.454	.2	.145	.8									
I 125	1	.035	1.																	
I 126	3	.86	.023	.48	.114	.386	.773													
I 128	4	.98	.009	.75	.004	.53	.009	.45	.16											
I 130	5	1.15	.235	.74	.765	.66	1.	.53	1.	.41	.235									
I 131	5	.724	.028	.638	.091	.364	.81	.284	.055	.08	.055									
XE125	5	.242	.5	.188	.25	.113	.25	.075	.025	.055	.5									
XE129M	2	.196	1.	.04	1.															
XE133	1	.081	1.																	
XE135	3	.61	.005	.36	.01	.25	1.													
XE137	1	0.	0.																	
CS132	3	1.2	.006	1.08	.006	.673	.99													
CS134	7	1.37	.05	1.17	.03	1.04	.06	.797	.84	.605	.98	.569	.26	.475	.04					
CS136	7	1.26	.21	1.07	.84	.83														

DY165M	1	.108	1.												
DY165	5	.715	.007	.634	.007	.362	.014	.28	.007	.095	.156				
HO166M	7	.83	.106	.81	.6	.751	.144	.711	.58	.28	.31	.184	.98	.081	1.
TM170	1	.084	.24												
YB169	7	.308	.11	.198	.49	.177	.34	.131	.25	.11	.59	.094	.15	.063	.55
LUI176M	1	.089	1.												
LUI177	5	.321	.002	.25	.002	.208	.11	.113	.058	.072	.001				
HF175	6	.432	.015	.343	.985	.319	.002	.229	.01	.114	.01	.089	.135		
HF180M	6	.501	.16	.444	.8	.332	.93	.215	.82	.093	.16	.058	.47		
HF181	7	.615	.003	.482	.83	.476	.02	.346	.14	.217	.005	.137	.04	.136	.15
TA180M	2	.102	.004	.093	.23										
TA182	7	1.23	.11	1.22	.28	1.19	.15	.222	.13	.122	.33	.1	.56	.068	.34
TA186	7	1.1	.1	.94	.15	.73	.75	.51	1.	.3	.25	.2	1.	.125	.25
W 185	1	0.56	.024												
W 187	6	.721	.01	.686	.3	.479	.3	.134	.31	.114	.01	.072	.3		
RE186	3	.768	.001	.137	.202	.123	.002								
RE188	6	1.96	.01	1.78	.001	1.67	.001	1.13	.01	.633	.01	.155	.2		
IR192M	1	.058	1.												
IR192	7	.79	.01	.6	.532	.485	.001	.417	.048	.375	.038	.302	1.8	.127	.097
IR194	7	.994	.002	.939	.014	.645	.027	.622	.01	.328	.252	.301	.01	.294	.033
PT193M	2	.135	1.	.013	1.										
PT197	3	.279	.01	.191	.09	.077	.99								
PT199	7	.96	.1	.79	.1	.475	.1	.318	.1	.246	.1	.197	.1	.074	.1
AU196	3	.426	.06	.356	.94	.333	.263								
AU198	3	1.09	.002	.675	.011	.412	1.								
AU199	3	.208	.16	.158	.77	.05	.08								
AU200	3	1.6	.01	1.23	.24	.368	.29								
HG197M	2	.164	.97	.133	.97										
HG197	2	.191	.02	.077	1.										
HG199M	2	.368	1.	.159	1.										
HG203	1	.279	1.												
TL202	3	.965	.002	.523	.002	.44	.996								
RI210	1	0.	0.												
PO210	1	.804	.001												
RA225	1	.04	.63												
TH228	4	.214	.5	.167	.01	.132	.029	.085	.29						
TH229	2	.2	.75	.148	.25										
TH231	5	.163	.02	.069	.2	.025	.13	.018	.2						
TH232	2	.79	.2	.059	1.										
TH233	5	.67	.003	.453	.01	.195	.01	.057	.01	.029	.02				
PA231	6														

APPENDIX C

SAMPLE PROBLEMS

The input data and the code output are presented below for three sample problems. These problems will include all of the features available in NAC. The three materials were analyzed in a single computer run and execution time was 0.14 minutes.

Each of the problems will use the same neutron flux:

Group 1 3.0×10^{12}

Group 2 1.5×10^{13}

Group 3 2.7×10^{13}

Group 4 4.0×10^{13}

Sample problem I: Aluminum 6063

Calculate the induced activity per gram for an irradiation period of 10 days with decay times of 0, 1 hour, 1 day, and 10 days

Composition (weight fraction):

Aluminum	.9765
Magnesium	.009
Silicon	.006
Iron	.0035
Copper	.001
Manganese	.001
Chromium	.001
Zinc	.001
Titanium	.001

Sample problem II: Stainless Steel 304L

Calculate the induced activity per cm^3 for an irradiation of 10 days with zero decay time. Perform gamma energy breakdown.

Density: 7.75 gram per-cubic centimeter

Composition (weight fraction):

Iron	.6412
Chromium	.200
Nickel	.12
Manganese	.02
Silicon	.01
Carbon	.008

Sample problem III: Experimental capsule

Calculate the induced activity for a total mass of 653.0 grams for an exposure of 5 cycles consisting of 10 days irradiation and 2 days non-irradiation with zero decay time.

Composition (weight fraction):

Tungsten	.151
Tantalum	.0583
Nickel	.495
Iron	.0743
Chromium	.0942
Aluminum	.127

INPUT DATA FOR THE THREE SAMPLE PROBLEMS, ILLUSTRATING THE CARD FORMAT

SAMPLE PROBLEM I ALUMINUM 6063												
1	9	13	12	14	26	29	25	24	30	22		
	1.0		.9765		.009		.006		.001		.001	.001
	.001		.001									
	3.0E+12		1.5E+13		2.7E+13		4.0E+13					
	1.44E+4		4									
	0.0E+0		6.0E+1		1.44E+3		1.44E+4					
SAMPLE PROBLEM II STAINLESS STEEL 304 L												
2	6	26	24	28	25	14	6					
	7.75		.6412		.20		.12		.01		.008	
	3.0E+12		1.5E+13		2.7E+13		4.0E+13					
	1.44E+4		1									
	0.0E+0											
SAMPLE PROBLEM III EXPERIMENTAL CAPSULE												
1	6	74	73	28	26	24	13					
	-653.		.151		.0583		.495		.0942		.127	
	3.0E+12		1.5E+13		2.7E+13		4.0E+13					
5	1.44E+4		2.88E+3		1							
	0.0E+0											

SAMPLE FC01EP I ALUMINUM 6063

NEUTRON FLUX 2.000E 12 1.500E 13 2.700E 14 4.000E 15 IRRADIATION TIME 1.440E 04 MIN

ELEMENT WEIGHT FRACTION WTS AT ZERO DECAY TIME ACTIVITY FRACTION PER NEUTRON GROUP

ALUMINUM C.9765C

AL 27 (N.G) AL 26	1.7874 11	0.	0.0001	C.0524	0.0269	0.9206
AL 27 (N.P) PG 27	3.5174 08	0.	1.0000	C.	0.	0.
AL 27 (N.A) NA 24	7.4774 07	0.	1.0000	C.	0.	0.
AL 27 (N.2N) AL 26	3.4014 01	0.	1.0000	C.	0.	0.

MAGNESIUM C.0050C

MG 24 (N.P) NA 24	1.1574 08	0.	1.0000	C.	0.	0.
MG 24 (N.G) PG 27	2.0174 07	0.	0.0000	C.0019	0.0283	0.9697

SILICON C.0060C

SI 28 (N.P) AL 26	4.1314 06	0.	1.0000	C.	0.	0.
SI 28 (N.P) AL 26	4.8674 04	0.	1.0000	C.	0.	0.
SI 28 (N.G) SI 21	1.3174 08	0.	0.0000	C.0000	0.0031	0.9969
SI 28 (N.A) MG 27	1.4474 05	0.	1.0000	C.	0.	0.

IRON C.0035C

FE 54 (N.P) PN 54	1.8874 04	0.	1.0000	C.	0.	0.
FE 54 (N.A) CR 51	5.4074 02	0.	1.0000	C.	0.	0.
FE 56 (N.P) PN 56	1.7874 05	0.	1.0000	C.	0.	0.
FE 56 (N.G) FE 55	1.0714 07	0.	0.0000	C.0000	0.0015	0.9985

COOPER C.0010C

CU 63 (N.G) CL 64	2.0074 06	0.	0.0001	C.0126	0.0460	0.9412
CU 63 (N.A) CO 60	5.0974 01	0.	1.0000	C.	0.	0.
CU 63 (N.2N) CL 62	4.5074 03	0.	1.0000	0.	0.	0.
CU 65 (N.G) CU 66	1.9074 08	0.	0.0002	0.0044	0.0795	0.9159
CU 65 (N.2N) CL 64	3.5674 03	0.	1.0000	C.	0.	0.

MANGANESE C.0010C

MN 55 (N.G) PN 56	5.0574 04	0.	0.0000	C.0011	0.0505	0.9484
MN 55 (N.A) V 52	6.3874 04	0.	1.0000	C.	0.	0.
MN 55 (N.2N) MN 54	4.2974 02	0.	1.0000	C.	0.	0.

CERMIUM C.0010C

CR 50 (N.G) CR 51
CR 50 (N.2N) CR 45
CR 52 (N.P) V 52

0.9713
0.
0.

0.0283
0.
0.

0.0004
0.
C.

0.0000
1.0000
1.0000

0.
0.
0.

0.
0.
0.

7.88+ 07
1.09+ 01
8.31+ 04

ZINC

C.CC1CC

ZN 64 (N.G) ZN 65
ZN 64 (N.P) CL 64
ZN 64 (N.2N) ZN 63
ZN 66 (N.P) CU 66
ZN 67 (N.P) CL 67
ZN 68 (N.G) ZN 65H
ZN 68 (N.A) NI 65
ZN 70 (N.G) ZN 71

0.9713
0.
0.
0.
0.
0.
0.9811
0.
0.9713

0.0283
0.
0.
0.
0.
0.0187
0.
0.0284

0.0004
C.
C.
C.
C.
C.0002
C.
C.0004

0.0000
1.0000
1.0000
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0.

1.95+ 06
9.46+ 04
5.22+ 07
2.51+ 04
2.84+ 02
8.49+ 06
1.02+ 02
1.63+ 05

TITANIUM

C.CC1CC

TI 46 (N.P) SC 46
TI 47 (N.P) SC 47
TI 48 (N.P) SC 48
TI 50 (N.G) TI 51
TI 50 (N.A) CA 47--SC 47

0.
0.
0.
0.9713
0.

0.
0.
0.
0.0283
0.

0.
C.
C.
C.0004
C.

1.0000
1.0000
1.0000
0.0000
1.0000

0.
0.
0.
0.
0.

0.
0.
0.
0.
2.069E-01

3.77+ 03
8.87+ 04
1.26+ 03
3.11+ 06
3.10+ 01

TIME AFTER IRRADIATION C.		MIN		ALL OUTPUT PER GRAM	
AL 28	4.5202E C3 PHILICLIFIES	4.0078E J3	MR/FR AT 1 M	3.1812E J1	MEV/SEC
PG 27	1.00045E C1 PHILICLIFIES	6.1040E J0	MR/FR AT 1 M	4.7969E J0	MEV/SEC
NA 24	5.1120E C0 PHILICLIFIES	1.0074E J1	MR/FR AT 1 M	7.8400E J0	MEV/SEC
AL 26	5.1864E-C7 PHILICLIFIES	8.2051E-J7	MR/FR AT 1 M	6.3853E J1	MEV/SEC
AL 25	1.2150E-C3 PHILICLIFIES	8.4084E-J0	MR/FR AT 1 M	6.5746E J0	MEV/SEC
SI 31	3.5555E C0 PHILICLIFIES	2.1087E-J03	MR/FR AT 1 M	1.6722E J5	MEV/SEC
PN 34	5.2100E-C4 PHILICLIFIES	2.0084E-J04	MR/FR AT 1 M	1.6056E J0	MEV/SEC
CR 31	2.1212E C0 PHILICLIFIES	3.0113E-J7	MR/FR AT 1 M	3.0438E J0	MEV/SEC
MN 36	1.3666E C2 PHILICLIFIES	1.1019E J7	MR/FR AT 1 M	8.9643E J0	MEV/SEC
FE 35	2.6555E-C1 PHILICLIFIES	1.0052E-J01	MR/FR AT 1 M	1.2725E J7	MEV/SEC
CU 44	5.4205E C0 PHILICLIFIES	1.5004E J0	MR/FR AT 1 M	1.1287E J0	MEV/SEC
CO 40	1.3755E-C6 PHILICLIFIES	1.0153E-J06	MR/FR AT 1 M	1.2726E J7	MEV/SEC
CU 42	1.2170E-C4 PHILICLIFIES	5.5065E-J7	MR/FR AT 1 M	4.3319E J1	MEV/SEC
CU 46	5.1414E C0 PHILICLIFIES	2.0086E-J01	MR/FR AT 1 M	1.8121E J7	MEV/SEC
V 52	3.5730E-C3 PHILICLIFIES	2.7011E-J03	MR/FR AT 1 M	2.1168E J5	MEV/SEC
CR 45	2.9562E-C7 PHILICLIFIES	8.0040E-J07	MR/FR AT 1 M	6.7269E-J1	MEV/SEC
ZN 45	5.2856E-C2 PHILICLIFIES	1.7083E-J02	MR/FR AT 1 M	9.5586E J5	MEV/SEC
ZN 43	1.4100E-C5 PHILICLIFIES	1.0074E-J06	MR/FR AT 1 M	8.3066E J1	MEV/SEC
CU 47	7.6517E-C6 PHILICLIFIES	4.3003E-J7	MR/FR AT 1 M	3.3932E J1	MEV/SEC
ZN 49M	2.2553E-C1 PHILICLIFIES	4.7000E-J2	MR/FR AT 1 M	3.7198E J0	MEV/SEC
NI 45	2.7737E-C6 PHILICLIFIES	9.0086E-J7	MR/FR AT 1 M	7.0650E J1	MEV/SEC
ZN 51	4.4110E-C3 PHILICLIFIES	1.0096E-J03	MR/FR AT 1 M	8.3235E J0	MEV/SEC
SC 46	1.0202E-C4 PHILICLIFIES	9.7098E-J05	MR/FR AT 1 M	7.5874E J0	MEV/SEC
SC 47	2.3580E-C3 PHILICLIFIES	1.7069E-J04	MR/FR AT 1 M	9.9371E J0	MEV/SEC
SC 48	3.4165E-C5 PHILICLIFIES	5.0055E-J05	MR/FR AT 1 M	4.2222E J0	MEV/SEC
TI 51	6.4045E-C2 PHILICLIFIES	1.0074E-J02	MR/FR AT 1 M	1.1186E J0	MEV/SEC
CA 47	8.4031E-C6 PHILICLIFIES	5.1066E-J09	MR/FR AT 1 M	4.0441E-J1	MEV/SEC
TOTAL MATERIAL ACTIVITY		4.5550E C3 PHILICLIFIES	4.2212E C3 MR/FR AT 1 M	3.0000E J1	MEV/SEC

TIME AFTER IRRADIATION		6.0000E 01 MIN	ALL OUTPUT PER uAM	
AL 26	7.3564E-05	PILLICLIFIES	6.7738E-03	MR/FR AT 1 M 4.8449E J3 MEV/SEC
PC 27	1.2044E-01	PILLICLIFIES	7.4053E-02	MR/FR AT 1 M 5.7473E J0 MEV/SEC
NA 24	4.5107E 00	PILLICLIFIES	9.0194E 00	MR/FR AT 1 M 7.4859E J0 MEV/SEC
AL 26	5.1844E-01	PILLICLIFIES	8.7751E-01	MR/FR AT 1 M 6.3853E J1 MEV/SEC
AL 25	7.6421E-06	PILLICLIFIES	1.0775E-06	MR/FR AT 1 M 1.3210E J2 MEV/SEC
SI 31	2.7863E 00	PILLICLIFIES	1.6425E-03	MR/FR AT 1 M 1.3093E J5 MEV/SEC
PN 54	5.2055E-04	PILLICLIFIES	2.0582E-04	MR/FR AT 1 M 1.6095E J4 MEV/SEC
CR 51	2.1250E 00	PILLICLIFIES	3.7172E-02	MR/FR AT 1 M 3.0406E J0 MEV/SEC
PN 56	1.0423E 02	PILLICLIFIES	8.7429E 01	MR/FR AT 1 M 6.8427E J3 MEV/SEC
FF 55	2.8525E-01	PILLICLIFIES	1.6542E-01	MR/FR AT 1 M 1.2717E J7 MEV/SEC
CU 64	5.1350E 00	PILLICLIFIES	1.4751E 00	MR/FR AT 1 M 1.0701E J0 MEV/SEC
CO 60	1.3757E-06	PILLICLIFIES	1.6552E-06	MR/FR AT 1 M 1.2726E J2 MEV/SEC
CU 62	1.8537E-06	PILLICLIFIES	8.4785E-04	MR/FR AT 1 M 6.5580E-01 MEV/SEC
CU 66	1.4784E-03	PILLICLIFIES	6.5566E-03	MR/FR AT 1 M 5.2114E J3 MEV/SEC
V 52	6.2242E-06	PILLICLIFIES	4.2014E-08	MR/FR AT 1 M 3.3162E J0 MEV/SEC
CR 45	1.0585E-07	PILLICLIFIES	3.2119E-04	MR/FR AT 1 M 2.4955E-01 MEV/SEC
ZN 65	5.2885E-02	PILLICLIFIES	1.2781E-02	MR/FR AT 1 M 9.5575E J3 MEV/SEC
ZN 63	4.4422E-06	PILLICLIFIES	3.3009E-07	MR/FR AT 1 M 2.6155E J1 MEV/SEC
CU 67	7.6042E-06	PILLICLIFIES	4.4107E-07	MR/FR AT 1 M 3.3546E J1 MEV/SEC
ZN 65M	2.1845E-01	PILLICLIFIES	4.5091E-02	MR/FR AT 1 M 3.5401E J0 MEV/SEC
AT 65	2.1246E-06	PILLICLIFIES	6.4737E-07	MR/FR AT 1 M 5.4115E J1 MEV/SEC
ZN 71	2.7150E-11	PILLICLIFIES	6.7432E-12	MR/FR AT 1 M 5.1231E-04 MEV/SEC
SC 46	1.0155E-04	PILLICLIFIES	9.7004E-05	MR/FR AT 1 M 7.5848E J3 MEV/SEC
SC 47	2.3777E-03	PILLICLIFIES	1.2561E-04	MR/FR AT 1 M 9.8531E J4 MEV/SEC
SC 48	3.2631E-05	PILLICLIFIES	5.4407E-05	MR/FR AT 1 M 4.1562E J3 MEV/SEC
TI 51	4.0583E-02	PILLICLIFIES	7.0194E-03	MR/FR AT 1 M 5.4548E J3 MEV/SEC
CA 47	6.2517E-05	PILLICLIFIES	5.1048E-04	MR/FR AT 1 M 4.0193E-01 MEV/SEC
TOTAL MATERIAL ACTIVITY		1.2000E 02 PILLICLIFIES	9.9267E 01 MR/FR AT 1 M	7.7250E 09 MEV/SEC

TIME AFTER IRRADIATION 1.440CE (3 MIN		ALL CUTEUT PER GRAM		
NA 24	1.6545E CC MILLICURIES	3.4231E 00 PR/FR AT 1 M	2.5861E 00 MEV/SEC	
AL 26	5.1824E-07 MILLICURIES	8.2051E-07 PR/FR AT 1 M	6.3853E 01 MEV/SEC	
SI 31	1.0032E-02 MILLICURIES	6.0050E-03 PR/FR AT 1 M	4.7161E 02 MEV/SEC	
PN 34	5.1555E-04 MILLICURIES	2.0038E-04 PR/FR AT 1 M	1.6061E 04 MEV/SEC	
CR 31	2.0707E CC MILLICURIES	3.0049E-07 PR/FR AT 1 M	2.9688E 00 MEV/SEC	
PN 36	2.0534E-01 MILLICURIES	1.7543E-01 PR/FR AT 1 M	1.3730E 02 MEV/SEC	
FE 35	2.8516E-01 MILLICURIES	1.8003E-01 PR/FR AT 1 M	1.2532E 02 MEV/SEC	
CU 44	1.5075E CC MILLICURIES	4.0039E-01 PR/FR AT 1 M	3.1392E 02 MEV/SEC	
CO 40	1.3753E-06 MILLICURIES	1.6547E-06 PR/FR AT 1 M	1.2721E 02 MEV/SEC	
CR 45	1.4187E-17 MILLICURIES	4.4084E-19 PR/FR AT 1 M	3.2283E-11 MEV/SEC	
ZN 45	5.2746E-02 MILLICURIES	1.2248E-02 PR/FR AT 1 M	9.5316E 02 MEV/SEC	
ZN 42	1.2722E-17 MILLICURIES	9.6256E-19 PR/FR AT 1 M	7.4908E-11 MEV/SEC	
CU 47	5.8422E-06 MILLICURIES	3.4147E-07 PR/FR AT 1 M	2.5795E 01 MEV/SEC	
ZN 45M	6.5545E-02 MILLICURIES	1.4567E-02 PR/FR AT 1 M	1.1336E 06 MEV/SEC	
AT 45	4.6118E-05 MILLICURIES	1.5094E-05 PR/FR AT 1 M	1.1746E-01 MEV/SEC	
SC 46	1.0118E-04 MILLICURIES	9.5597E-05 PR/FR AT 1 M	7.5250E 03 MEV/SEC	
SC 47	1.9556E-03 MILLICURIES	1.0014E-03 PR/FR AT 1 M	8.1042E 06 MEV/SEC	
SC 48	2.3408E-05 MILLICURIES	3.7172E-05 PR/FR AT 1 M	2.8927E 03 MEV/SEC	
TI 51	2.7458E-05 MILLICURIES	4.6501E-05 PR/FR AT 1 M	3.6546E-02 MEV/SEC	
CA 47	7.2211E-05 MILLICURIES	4.6642E-05 PR/FR AT 1 M	3.4896E-01 MEV/SEC	
TOTAL MATERIAL ACTIVITY		5.9125E CC MILLICURIES	4.1254E 00 GR/HR AT 1 M	3.2135E 08 MEV/SEC

TIME AFTER IRRADIATION 1.4400E 04 MIN		ALL OUTPUT PER GRAM	
NA 24	7.4434E-05 MILLICURIES	1.7334E-04 MR/HR AT 1 M	1.1557E 04 MEV/SEC
AL 26	5.1864E-07 MILLICURIES	8.7351E-07 MR/HR AT 1 M	6.3853E 01 MEV/SEC
SI 31	1.1333E-25 MILLICURIES	6.0434E-09 MR/HR AT 1 M	5.3256E-21 MEV/SEC
PN 34	5.0501E-04 MILLICURIES	2.0032E-04 MR/HR AT 1 M	1.5745E 04 MEV/SEC
CR 37	1.6005E 00 MILLICURIES	3.0475E-07 MR/HR AT 1 M	2.3716E 00 MEV/SEC
PN 46	5.7008E-27 MILLICURIES	8.1424E-27 MR/HR AT 1 M	6.3676E-19 MEV/SEC
FF 49	2.4437E-01 MILLICURIES	1.4425E-01 MR/HR AT 1 M	1.0915E 07 MEV/SEC
CU 64	1.5006E-05 MILLICURIES	4.0052E-06 MR/HR AT 1 M	3.1247E 02 MEV/SEC
CO 60	1.3768E-06 MILLICURIES	1.4794E-06 MR/HR AT 1 M	1.2680E 02 MEV/SEC
ZN 65	5.1417E-02 MILLICURIES	1.4339E-02 MR/HR AT 1 M	5.2914E 03 MEV/SEC
CU 67	4.5978E-07 MILLICURIES	2.6005E-04 MR/HR AT 1 M	2.1871E 00 MEV/SEC
ZN 69M	1.5858E-06 MILLICURIES	3.4318E-07 MR/HR AT 1 M	2.5695E 01 MEV/SEC
NI 66	4.4732E-34 MILLICURIES	1.4554E-14 MR/HR AT 1 M	1.1404E-20 MEV/SEC
SC 46	5.3543E-05 MILLICURIES	8.4777E-05 MR/HR AT 1 M	6.9866E 03 MEV/SEC
SC 47	3.1211E-04 MILLICURIES	1.4420E-05 MR/HR AT 1 M	1.2934E 03 MEV/SEC
SC 48	7.7842E-07 MILLICURIES	1.7654E-06 MR/HR AT 1 M	9.6222E 01 MEV/SEC
CA 47	1.5233E-05 MILLICURIES	1.4494E-04 MR/HR AT 1 M	9.2561E-07 MEV/SEC
TOTAL MATERIAL ACTIVITY 1.5613E 00 MILLICURIES		1.8314E-01 MR/HR AT 1 M	1.4422E 07 MEV/SEC

SAMPLE PROBLEM II STAINLESS STEEL 304L

NEUTRON FLUX 2.00CE 12 1.50CE 13 2.710E 13 4.000E 13 IRRADIATION TIME 1.440E 04 MIN

ELEMENT IS NOT LISTED IN LIBRARY Z= 6

ELEMENT	WEIGHT FRACTION	1/25 AT /ERC DECAY TIME	ACTIVITY FRACTION PER NEUTRON GROUP
IRON			
C.6412C			
FE 54 (N.P) MN 54	2.67E+07	0.	1.0000 0.
FE 54 (N.A) CR 51	7.67E+05	0.	1.0000 0.
FE 52 (N.P) MN 52	2.53E+08	0.	1.0000 0.
FE 52 (N.G) FE 55	1.52E+10	0.	0.0000 0.0015 0.9985
CHROMIUM			
C.2000C			
CR 50 (N.G) CR 51	1.22E+11	0.	0.0000 0.0283 0.9713
CR 50 (N.2N) CR 45	1.69E+04	0.	1.0000 0.
CR 52 (N.P) V 52	1.28E+08	0.	1.0000 0.
NICKEL			
C.1200C			
NI 58 (N.P) CO 58	2.52E+08	2.18E-07	1.0000 0.
NI 58 (N.P) CO 58	2.32E+08	0.	1.0000 0.
NI 58 (N.P) CO 57	2.10E+07	0.	1.0000 0.
NI 58 (N.2N) NI 57	1.20E+05	0.	1.0000 0.
NI 60 (N.P) CO 60	1.34E+05	0.	1.0000 0.
NI 62 (N.A) FE 55	2.08E+04	0.	1.0000 0.
NI 64 (N.G) NI 65	5.60E+04	0.	0.0004 0.0284 0.9713
MANGANESE			
C.0200C			
MN 55 (N.G) MN 56	7.83E+11	0.	0.0000 0.0505 0.9484
MN 55 (N.A) V 52	9.89E+06	0.	1.0000 0.
MN 55 (N.2N) MN 54	6.65E+04	0.	1.0000 0.
SILICON			
C.0100C			
SI 28 (N.P) AL 28	5.33E+07	0.	1.0000 0.
SI 25 (N.P) AL 25	6.28E+05	0.	1.0000 0.
SI 20 (N.G) SI 21	1.70E+04	0.	0.0000 0.0031 0.9969
SI 20 (N.A) MG 27	1.87E+06	0.	1.0000 0.

TIME AFTER IRRADIATION C. PIN ALL OUTPUT PER LOGIC CENTIMETER

PN 54	3.25C3E-C1 MILLICLIFES	2.8084E-01	MR/FR AT 1 M	2.2400E 01	MEV/SEC		
	GAMMA ENERGY	C.8350					
	MR/FR AT 1 M	2.8784E-C1					
	MEV/SEC	2.2400E 01					
CR 51	3.30C4E C3 MILLICLIFES	6.0524E 01	MR/FR AT 1 M	4.7175E 01	MEV/SEC		
	GAMMA ENERGY	C.6500					
	MR/FR AT 1 M	1.0205E C1	0.4500	0.3200			
	MEV/SEC	1.5446E C6	5.1824E 00	4.5233E 01			
		2.1151E C4 MILLICLIFES	4.0334E 00	3.5201E 09			
			1.7500E 01	MR/FR AT 1 M	1.3895E 01	MEV/SEC	
PN 56	GAMMA ENERGY	2.3500		2.6600			
	MR/FR AT 1 M	6.8311E C1	1.1924E 02	1.8760E 02	2.5200	2.1200	1.8100
	MEV/SEC	5.3161E C5	9.2834E 09	1.4600E 10	3.0468E 02	3.7001E 03	5.3980E 03
					2.3711E 10	2.5432E 11	4.2008E 11
PF 55	4.1114E C2 MILLICLIFES	2.3717E 02	MR/FR AT 1 M	1.8068E 10	MEV/SEC		
	GAMMA ENERGY	1.2500					
	MR/FR AT 1 M	1.0843E C2	1.1000	0.2000			
	MEV/SEC	6.4382E C5	1.2256E 02	1.1729E 00			
		4.5821E-C4 MILLICLIFES	9.5384E 04	9.1273E C7			
			1.1594E-05	MR/FR AT 1 M	1.0427E 01	MEV/SEC	
CR 45	GAMMA ENERGY	C.1500		0.0600			
	MR/FR AT 1 M	4.5750E-C6	6.8624E-06	1.9607E-C6			
	MEV/SEC	3.5403E C2	5.3604E 02	1.5259E 02	1.9986E 00	MEV/SEC	
		3.7512E C0 MILLICLIFES	2.7582E 00	MR/FR AT 1 M			
V 52	GAMMA ENERGY	1.4400					
	MR/FR AT 1 M	2.5682E C0					
	MEV/SEC	1.5566E C8					
CO 58P	6.6151E C0 MILLICLIFES	8.1004E-02	MR/FR AT 1 M	6.3040E 00	MEV/SEC		
	GAMMA ENERGY	C.0250					
	MR/FR AT 1 M	8.1004E-C2					
	MEV/SEC	6.3040E C6					
CC 58	GAMMA ENERGY	1.6500		0.8050			
	MR/FR AT 1 M	2.7003E-C2	4.2414E-02	2.6216E 00			
	MEV/SEC	2.1014E C6	3.3011E 06	2.0402E C8	2.8835E 00	MEV/SEC	
		5.6826E-C1 MILLICLIFES	3.7153E-02	MR/FR AT 1 M			
CO 57	GAMMA ENERGY	C.7070		0.1000			
	MR/FR AT 1 M	3.8203E-C4	3.6670E-02				
	MEV/SEC	2.5730E C4	2.8534E 06		1.7561E 01	MEV/SEC	
		3.2246E-C3 MILLICLIFES	2.7506E-01	MR/FR AT 1 M			
AT 57	GAMMA ENERGY	1.9000		0.1270			
	MR/FR AT 1 M	4.1058E-C4	1.8184E-03	2.7444E-C5			
	MEV/SEC	3.1552E C4	1.4154E 05	2.1357E C3			
		2.6255E-C3 MILLICLIFES	4.1558E-01	MR/FR AT 1 M	3.3664E 01	MEV/SEC	
CO 60	GAMMA ENERGY	1.3300		1.1000			
	MR/FR AT 1 M	2.3013E-C3	2.0244E-03				
	MEV/SEC	1.7505E C5	1.5755E 05				

NI 65	1.5154E C7 PHILICLIFIES	4.9700E J1	MR/FR AT 1 M	3.860CE J9 MEV/SEC
	GAMMA ENERGY	1.46CC		
	MR/FR AT 1 M	1.1090E J1	0.2700	
	MEV/SEC	2.3741E C5	7.9977E 00	
		1.4421E C6 PHILICLIFIES	8.6350E 08	
AL 28		1.7705E J3	MR/FR AT 1 M	9.4979E J7 MEV/SEC
	GAMMA ENERGY	1.78CC		
	MR/FR AT 1 M	1.7205E C6		
	MEV/SEC	5.4978E C7		
	1.6685E-C7 PHILICLIFIES	1.7712E-J2	MR/FR AT 1 M	8.4922E J5 MEV/SEC
	GAMMA ENERGY	2.43CC		
	MR/FR AT 1 M	1.2107E-C3		
	MEV/SEC	5.4482E C4		
	4.5664E C1 PHILICLIFIES	2.7754E-J2	MR/FR AT 1 M	2.1599E J0 MEV/SEC
SI 31				
	GAMMA ENERGY	1.27CC		
	MR/FR AT 1 M	2.7754E-C2		
	MEV/SEC	2.1555E C6		
	5.0524E-C2 PHILICLIFIES	3.3997E-J2	MR/FR AT 1 M	2.4122E J0 MEV/SEC
MG 27				
	GAMMA ENERGY	1.02CC		
	MR/FR AT 1 M	1.0538E-C2		
	MEV/SEC	8.2000E C5		
		0.8140	1.7500	
		2.0034E-02	4.2046E-04	
		1.5590E 06	3.2721E C4	
TOTAL MATERIAL ACTIVITY	2.5124E C4 PHILICLIFIES	1.8209E 04 MR/HR	AT 1 M	1.4170E 12 MEV/SEC

SAMPLE FOLDER III EXPERIMENTAL CAPSULE

NEUTRON FLUX	3.00E 12	1.50E 13	2.00E 14	4.00E 15	CYCLES	IRRADIATION TIME 1.440E 04 MIN
ELEMENT	WEIGHT FRAC (G)	WPS AT ZERO DECAY TIME				
TUNGSTEN C.1510C						
W 184 (N.G.) W 185	3.57E 12	0.	0.	0.	0.0000	0.3473 0.6227
W 186 (N.G.) W 187	1.07E 14	0.	0.	0.	0.0000	0.0015 0.9563
W 188 (N.P.) W 189	1.95E 05	0.	0.	0.	1.0000	0. 0.
TANTALUM C.0583C						
TA 181 (N.G.) TA 182	4.27E 13	0.	0.	0.	0.0000	0.0082 0.4949 0.4970
TA 181 (N.2N) TA 182	5.13E 06	0.	0.	0.	1.0000	0. 0.
NICKEL C.4550C						
NI 58 (N.P.) CO 58N--CC 58	8.76E 10	7.60E 09	0.	0.	1.0000	0. 0.
NI 58 (N.P.) CO 59	3.25E 11	0.	0.	0.	1.0000	0. 0.
NI 58 (N.P.) CO 57	3.43E 10	0.	0.	0.	1.0000	0. 0.
NI 58 (N.2N) NI 57	4.19E 07	0.	0.	0.	1.0000	0. 0.
NI 60 (N.P.) CO 60	2.32E 08	0.	0.	0.	1.0000	0. 0.
NI 62 (N.A) FE 55	2.59E 07	0.	0.	0.	1.0000	0. 0.
NI 64 (N.G) NI 65	1.94E 12	0.	0.	0.	0.0000	0.0004 0.0284 0.9713
IRON C.0743C						
FE 54 (N.P.) MN 54	1.24E 09	0.	0.	0.	1.0000	0. 0.
FE 54 (N.A) CR 51	2.24E 07	0.	0.	0.	1.0000	0. 0.
FE 56 (N.P.) MN 56	2.47E 04	0.	0.	0.	1.0000	0. 0.
FE 58 (N.G) FE 55	5.31E 11	0.	0.	0.	0.0000	0.0005 0.9985
CHROMIUM C.0542C						
CR 50 (N.G) CR 51	1.45E 13	0.	0.	0.	0.0000	0.0004 0.0283 0.9713
CR 50 (N.2N) CR 45	6.72E 05	0.	0.	0.	1.0000	0. 0.
CR 52 (N.P.) V 52	5.11E 09	0.	0.	0.	1.0000	0. 0.
ALUMINUM C.1270C						
AL 27 (N.G) AL 28	1.51E 13	0.	0.	0.	0.0000	0.0024 0.0269 0.9206
AL 27 (N.P.) PG 27	2.98E 10	0.	0.	0.	1.0000	0. 0.
AL 27 (N.A) NA 24	6.34E 04	0.	0.	0.	1.0000	0. 0.
AL 27 (N.2N) AL 26	1.35E 04	0.	0.	0.	1.0000	0. 0.

TIME AFTER IRRADIATION C.		MIN		ALL OUTPUT FOR TOTAL MASS	
W 1E	5.6137E C4 MILLICURIES	6.1515E 32	MR/FR AT 1 M	4.8105E 10	MEV/SEC
W 1F	2.9111E C6 MILLICURIES	5.8709E 05	MR/FR AT 1 M	4.5345E 15	MEV/SEC
TA 1E	5.2782E-C3 MILLICURIES	4.0522E-34	MR/FR AT 1 M	3.1535E 05	MEV/SEC
TA 1F	1.1500E C6 MILLICURIES	4.4172E 35	MR/FR AT 1 M	2.4375E 15	MEV/SEC
TA 1G	1.3881E C1 MILLICURIES	1.4586E-31	MR/FR AT 1 M	1.1195E 07	MEV/SEC
CO 1E	2.3887E C3 MILLICURIES	2.4055E 01	MR/FR AT 1 M	2.1910E 04	MEV/SEC
CO 1F	5.0045E C3 MILLICURIES	3.5709E 03	MR/FR AT 1 M	2.7400E 11	MEV/SEC
CO 1G	5.2540E C2 MILLICURIES	6.0501E 31	MR/FR AT 1 M	4.7160E 04	MEV/SEC
NI 1F	1.1340E C0 MILLICURIES	7.8826E-31	MR/FR AT 1 M	6.1343E 17	MEV/SEC
CO 1C	6.2702E C0 MILLICURIES	7.8529E 03	MR/FR AT 1 M	5.7999E 08	MEV/SEC
FE 1F	1.4342E C4 MILLICURIES	8.1101E 03	MR/FR AT 1 M	6.3114E 11	MEV/SEC
NI 1E	5.2671E C4 MILLICURIES	1.7739E 04	MR/FR AT 1 M	1.3416E 17	MEV/SEC
PN 1A	3.3507E C1 MILLICURIES	1.5502E 01	MR/FR AT 1 M	1.0352E 04	MEV/SEC
CR 1F	3.9321E C5 MILLICURIES	7.2104E 03	MR/FR AT 1 M	5.6155E 11	MEV/SEC
PN 1C	6.6707E C1 MILLICURIES	5.8276E 01	MR/FR AT 1 M	4.3810E 04	MEV/SEC
CR 1E	1.8144E-C2 MILLICURIES	5.4172E-34	MR/FR AT 1 M	4.1375E 04	MEV/SEC
V 1F	1.3825E C2 MILLICURIES	9.4555E 01	MR/FR AT 1 M	7.3662E 04	MEV/SEC
AL 1E	4.1015E C5 MILLICURIES	3.4715E 05	MR/FR AT 1 M	2.7015E 15	MEV/SEC
MG 1F	8.0050E C2 MILLICURIES	4.4073E 32	MR/FR AT 1 M	3.8516E 10	MEV/SEC
NA 1A	1.7100E C2 MILLICURIES	3.4513E 32	MR/FR AT 1 M	2.6158E 10	MEV/SEC
AL 1G	3.6685E-C4 MILLICURIES	3.7702E-34	MR/FR AT 1 M	2.5496E 04	MEV/SEC
TOTAL MATERIAL ACTIVITY		5.0475E C6 MILLICURIES	1.4093E C6 MR/HR AT 1 M	1.04408E 14	MEV/SEC

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